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VULCAN - A DEPLETION PROGRAM FOR USE WITH INDEPENDENT ONE-DIMENSIONAL SPATIAL CALCULATIONS

by John L. Anderson, Jr.

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SUMMARY

A digital computer depletion program, VULCAN, which treats the time behavior of nuclide concentrations in a nuclear reactor, is described. This depletion program is suitable for use with any multigroup one-dimensional spatial solution; that is, it is independent of the actual generating source of cross sections or fluxes. This flexibility allows any space-energy calculational sequence to be extended, without alteration, to include a depletion stage.

The depletion calculation is performed for each mesh interval, and an averaging process is used to reduce the number of regions for which new atom densities are required. The depletion equations for several classes of nuclides are shown and cast into forms solvable by a computer. Data input instructions, a sample problem, and a FORTRAN IV listing of the program are included. VULCAN, written for an IBM 7094, relies entirely on fast memory computer storage (32K).

INTRODUCTION

In the face of various economic and operational restrictions the life expectancy of a nuclear reactor must be considered as a part of its design. This life expectancy or lifetime is essentially the length of time a reactor core or fuel element may operate before reactivity or structural considerations force a shutdown. For example, the structural consideration may be the governing one because of gaseous fission products and consequent pressure buildup.

In the interest of economics the degree of fuel utilization within a spent element or core should be as large as possible. Budgetary requirements and, for commercial power reactors, subsequent consumer costs make necessary the availability of a cost per unit time figure which depends on the fuel utilization. In another vein, space applications of nuclear reactors, for which maintenance is rather inconvenient, demand

lifetime and performance foreknowledge.

The lifetime determination based on reactivity considerations may be made by considering the time behavior of fissionable and parasitic nuclide concentrations. The behavior of individual nuclides is governed by depletion (productive and parasitic absorption, decay and physical losses) and by buildup (decay and fission sources). The differential equations describing the various isotopic populations are typically solved by programs for digital computers; generically, these programs are called depletion programs.

Depletion programs may emphasize any of several facets: speed, intervention convenience, tape handling convenience, and calculational detail. Usually, the programs include the spatial (generally diffusion theory) calculations and cross section libraries in one all-encompassing computer program. These programs may treat reactors whose symmetry allows their calculation to be performed in one (refs. 1 and 2), two (refs. 3 and 4), or three dimensions. They allow several depletion cycles to be performed without any other input or intervention. However, such codes typically have energy group and diffusion theory limitations because of insufficient computer memory capacity. These limitations may be overcome by auxiliary storage on peripheral devices but at the expense of computer execution time. A somewhat more comprehensive review of the literature and programs may be found in reference 5.

This depletion program, VULCAN, may be used with any one-dimensional space-energy solution. The cross sections and fluxes (mesh interval averages) that VULCAN requires may be from any source (constant, analytical, or even experimental) in any format. Consequently VULCAN is independent of the actual generating source of cross sections or fluxes. This flexibility allows any space-energy calculational sequence to be extended without alteration to include a depletion stage. A particular advantage resulting from the independent status of VULCAN is the availability of considerably more fast memory computer storage since the multigroup energy and spatial solutions to Boltzmann's equation have been obtained from other programs. This additional storage enables pointwise (at each discrete mesh interval of the spatial calculation) rather than zonal (average over several mesh intervals) nuclide concentrations to be followed. A representative 7 energy group-14 mesh interval-13 isotope-1 time step problem required 0.15 minute of IBM 7094-II computer execution time.

The VULCAN program is comprised of about 1200 source statements and relies entirely on fast memory computer storage. However, because no peripheral storage is used, the following restrictions are imposed on this version of VULCAN:

Nuclides	≤40
Energy groups	≤40
Mesh intervals	≤20
Zones	≤190
Materials	≤190

Some of the restrictions may be relaxed but at the expense of tightening others (see the section VULCAN DESCRIPTION). VULCAN is written in FORTRAN IV language for use on an IBM 7094 with 32K memory.

PREPARATION OF DEPLETION EQUATIONS

General Solutions and Approximations

The general equation governing the concentration of a particular isotope N as a function of space \vec{r} , time t , and energy E is

$$\frac{dN(\vec{r}, t)}{dt} = \sum_i^{NF} \Gamma_i^N N_i(\vec{r}, t) \int_0^\infty \Phi(\vec{r}, E, t) \sigma_f^i(E) dE + \lambda_p N_p(\vec{r}, t) + N_r(\vec{r}, t) \int_0^\infty \Phi(\vec{r}, E, t) \sigma_{n,\gamma}^r(E) dE - \lambda_N N(\vec{r}, t) - N(\vec{r}, t) \int_0^\infty \Phi(\vec{r}, E, t) \sigma_a^N(E) dE - \xi_N N(\vec{r}, t) \quad (1)$$

where

$\Phi(\vec{r}, E, t)$	neutron flux as function of space, energy, and time, neutrons/(b) (sec); neutrons/(10^{-28} m^2) (sec)
$\sigma_f^k, \sigma_{n,\gamma}^k, \sigma_a^k$	microscopic fission, (n, γ), and absorption cross sections for isotope k , b; 10^{-28} m^2
λ_k	decay constant for isotope k , sec^{-1}
N_k	atom concentration of isotope k , atoms/(b) (cm); atoms/(10^{-30} m^3)
NF	number of fissionable isotopes considered

The first term on the right side of the equation is the source of isotope N from the fission yield Γ_i^N of each fissionable isotope i considered. The second term is the

source from beta decay of the precursor nuclide N_p . The third term is the source from radiative capture (n, γ reaction) by N_x . The fourth term is the loss resulting from beta decay of the isotope N itself. The fifth term is the loss from depletion or burnup of N . The sixth term represents losses of isotope N such as expulsion of material fragments or diffusion of gaseous constituents or products (ξ_N is the loss rate).

There are two simplifications to be considered before this generalized equation may be used in a depletion scheme. The first is the reduction from the continuous variables \vec{r} , E , and t to their discrete counterparts. In particular, the GAM-GATHER (refs. 6 and 7) multigroup energy structure and the TDSN (ref. 8) spatial interval structure may establish the discrete E and \vec{r} . There remains only the discrete t to assign, and this is done as part of the VULCAN input.

The second simplification begins with the realization that the form of equation (1) and its consequent solution will depend on the nuclear behavior of the isotope. Contrast, for example, the form of the equation for uranium 235 (U^{235}) which is fissionable and may be formed by radiative capture in U^{234} and the form for xenon 135 (Xe^{135}) which is a fission product and is both created and depleted by beta decay.

Now, before enumerating the forms of the depletion equation to be considered, it is convenient to discuss some methods for their solution. There are two assumptions in general use which facilitate the solutions: (1) constant flux or (2) constant power over a time interval, each of which will be expressed mathematically hereinafter. As the time interval decreases, the two approximations approach each other. However, the constant power approximation (CPA) seems to be appropriate since the usually understood objective of the reactors considered is that of providing a constant power output. Since most of the power in a reactor is produced by any of six isotopes (thorium 232 (Th^{232}), U^{233} , U^{235} , U^{238} , plutonium 239 (Pu^{239}), and Pu^{241}), the CPA need only be applied to equations which dictate their behavior. The equations for all other isotopes may be treated by the constant flux approximation and then solved analytically.

At this stage it is appropriate to cast equation (1) in its discrete form, that is,

$$\int_0^\infty \Phi(\vec{r}, E, t) \sigma_x^N(E) dE$$

becomes

$$\sum_j^{NG} \Phi_{jm}(t) \sigma_{x_j}^N$$

for any isotope N , mesh interval (spatial location) m , cross section σ_x , and time t and where NG is the total number of energy groups. Thus,

$$\left. \frac{dN_m(t)}{dt} \right]_m = \sum_i^{NF} \Gamma_i^N N_{im}(t) \sum_j^{NG} \Phi_{jm}(t) \sigma_{fj}^i + \lambda_p N_{pm}(t) + N_{rm}(t) \sum_j^{NG} \Phi_{jm}(t) \sigma_{n,\gamma_j}^r - \lambda_N N_m(t) - N_m(t) \sum_j^{NG} \Phi_{jm}(t) \sigma_{a_j}^N - \xi_N N_m(t) \quad (2)$$

In the CPA, $N_{im}(t) \sum_j^{NG} \Phi_{jm} \sigma_{fj}^i$ is a constant for any mesh interval m where i corresponds to the six isotopes mentioned previously. In the constant flux assumption, Φ_{jm} is considered constant over the time interval, and $N_m(t)$ is thus free to be combined with $dN_m(t)$ leading to an analytic solution. Of course, in either case, Φ_{jm} and $\sigma_{x_j}^i$ are implicitly constant over the energy j and space m intervals as is required by the respective codes. For the finite difference solution to the CPA, the following two substitutions must be made:

$$\frac{dN(t)}{dt} \rightarrow \frac{N_{t+\Delta t} - N_t}{\Delta t}$$

and

$$N(t) \rightarrow \frac{N_{t+\Delta t} + N_t}{2}$$

Formulation of Specific Equations

It now becomes necessary to determine the isotopes which will be explicitly considered. A fixed base of twelve fissionable isotopes and five assorted fission products was established, and the basic depletion equations determined. These equations and two other general forms are considered adequate for any foreseeable reactor application.

The twelve fissionable isotopes used as a base are the constituent nuclides in two chains $\text{Th}^{232} \rightarrow \text{U}^{236}$ and $\text{U}^{238} \rightarrow \text{Pu}^{242}$. The other five base isotopes are Xe^{135} , samarium 149 (Sm^{149}), iodine 135 (I^{135}), promethium 149 (Pm^{149}), and a fission product aggregate; I^{135} and Pm^{149} are considered merely because they are precursors of Xe^{135} and Sm^{149} , which are major parasitic absorbers. Four additional categories of isotopes may also be treated. The first of these categories is labeled "Special

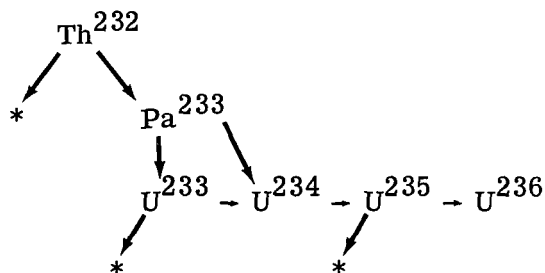
Materials'' and may include additional fission products and aggregates and neutron capture products. These isotopes are formed from fission and/or capture and decay (see the section INPUT INSTRUCTIONS). The analytic solution is used and so is applicable to rapidly saturating nuclides. The second is headed ''Burnable Poisons'' and treats isotopes depleted by absorption and those which are not continuously formed (e.g., boron 10). The third category, called ''Repeated Fissionable Nuclides'' allows any fissionable isotope to be included more than once. If, for example, U^{235} occurs in two locations in a reactor where the flux spectra differ, the microscopic cross sections may be sufficiently different to make separate depletion desirable. The fourth category is headed ''Nondepletable Nuclides'' and it allows nondepletable isotopes such as hydrogen, oxygen, or aluminum to be carried through the VULCAN calculation unaltered.

The following two sections contain the specific depletion equations considered. Their solutions, in a form suitable for computer analysis, are shown along with some exemplary, but condensed, derivations. In the equations that follow, any $\sigma_x^i \Phi$ implies a type of summation convention, that is,

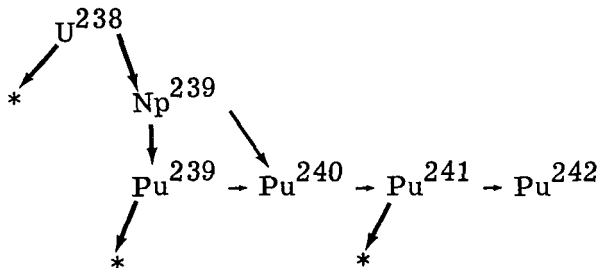
$$\sum_j^{NG} \sigma_{x_j}^i \Phi_j \rightarrow \sigma_{x_j}^i \Phi_j \rightarrow \sigma_x^i \Phi$$

Fissionable nuclides. - The following two simplified heavy element chains provided the twelve basic fissionable nuclides:

$Th^{232} \rightarrow U^{236}$ chain:



$U^{238} \rightarrow Pu^{242}$ chain:



where

- \searrow (n, γ) process followed by β decay
- \downarrow β decay
- \rightarrow (n, γ) reaction
- $\star \swarrow$ fission (all isotopes fission but virtually all power is produced by the six indicated)

The two chains appear to obey identical reaction mechanisms. To the extent of the considerations in this depletion analysis, this is the case, with but one exception; Pu²⁴¹ undergoes β decay with a nonnegligible decay whereas U²³⁵, for these depletion purposes, is stable. Consequently, seven equations suffice to treat the twelve fissionable nuclides.

In the following equations the superscripts represent the VULCAN internal identification (VID) number (see the section Nuclide List) for the first chain considered (Th²³² \rightarrow U²³⁶). The isotope in the second chain (U²³⁸ \rightarrow Pu²⁴²) to which each equation applies is also indicated. The equation designation (a) will denote the differential equation and (b) will denote the finite difference form used in the program.

Th²³² (VID = 1) or U²³⁸ (VID = 7)

$$\frac{dN^1}{dt} = -N^1 \sigma_a^1 \Phi \quad (3a)$$

$$\frac{N_{t+\Delta t}^1 - N_t^1}{\Delta t} = - \left(\frac{N_{t+\Delta t}^1 + N_t^1}{2} \right) \sigma_a^1 \Phi$$

$$N_{t+\Delta t}^1 = N_t^1 \left(\frac{2 - \sigma_a^1 \Phi \Delta t}{2 + \sigma_a^1 \Phi \Delta t} \right) \quad (3b)$$

Pa²³³ (VID = 2) or Np²³⁹ (VID = 8)

$$\frac{dN^2}{dt} = N^1 \left(\sigma_a^1 - \sigma_f^1 \right) \Phi - N^2 \left(\lambda^2 + \sigma_a^2 \Phi \right) \quad (4a)$$

$$\left\{ \ln \left[N^2 \left(\lambda^2 + \sigma_a^2 \Phi \right) - N^1 \left(\sigma_a^1 - \sigma_f^1 \right) \Phi \right] \right\}_{N_t^2}^{N_{t+\Delta t}^2} = - \left(\lambda^2 + \sigma_a^2 \Phi \right) \Delta t$$

$$N_{t+\Delta t}^2 = \frac{N_{t+\Delta t}^1 + N_t^1}{2} \frac{(\sigma_a^1 - \sigma_f^1) \Phi}{\lambda^2 + \sigma_a^2 \Phi} \left\{ 1 - \exp \left[- \left(\lambda^2 + \sigma_a^2 \Phi \right) \Delta t \right] \right\} + N_t^2 \exp \left[- \left(\lambda^2 + \sigma_a^2 \Phi \right) \Delta t \right] \quad (4b)$$

$$U^{233} \text{ (VID = 3) or Pu}^{239} \text{ (VID = 9)}$$

$$\frac{dN^3}{dt} = - N^3 \sigma_a^3 \Phi + N^2 \lambda^2 + \frac{N_{t+\Delta t}^3 - N_t^3}{\Delta t} = - \left(\frac{N_{t+\Delta t}^3 + N_t^3}{2} \right) \sigma_a^3 \Phi + N^2 \lambda^2 \quad (5a)$$

$$N_{t+\Delta t}^3 = \frac{N_t^3 (2 - \sigma_a^3 \Phi \Delta t) + (N_{t+\Delta t}^2 + N_t^2) \lambda^2 \Delta t}{2 + \sigma_a^3 \Phi \Delta t} \quad (5b)$$

If VID = 2 (VID = 8) is not included, then the following equation is used:

$$N_{t+\Delta t}^3 = \frac{N_t^3 (2 - \sigma_a^3 \Phi \Delta t) + (N_{t+\Delta t}^1 + N_t^1) (\sigma_a^1 - \sigma_f^1) \Phi \Delta t}{2 + \sigma_a^3 \Phi \Delta t} \quad (5c)$$

$$U^{234} \text{ (VID = 4) or Pu}^{240} \text{ (VID = 10)}$$

$$\frac{dN^4}{dt} = - N^4 \sigma_a^4 \Phi + N_t^2 (\sigma_a^2 - \sigma_f^2) \Phi + N^3 (\sigma_a^3 - \sigma_f^3) \Phi \quad (6a)$$

$$N_{t+\Delta t}^4 = N_t^4 e^{-\sigma_a^4 \Phi \Delta t} + \left[(N_{t+\Delta t}^2 + N_t^2) (\sigma_a^2 - \sigma_f^2) + (N_{t+\Delta t}^3 + N_t^3) (\sigma_a^3 - \sigma_f^3) \right] \left(\frac{1 - e^{-\sigma_a^4 \Phi \Delta t}}{2 \sigma_a^4} \right) \quad (6b)$$

U^{235} (VID = 5)

$$\frac{dN^5}{dt} = -N^5 \sigma_a^5 \Phi + N^4 \left(\sigma_a^4 - \sigma_f^4 \right) \Phi \quad (7a)$$

$$N_{t+\Delta t}^5 = \frac{N_t^5 \left(2 - \sigma_a^5 \Phi \Delta t \right) + \left(N_{t+\Delta t}^4 + N_t^4 \right) \left(\sigma_a^4 - \sigma_f^4 \right) \Phi \Delta t}{2 + \sigma_a^5 \Phi \Delta t} \quad (7b)$$

Pu^{241} (VID = 11)

$$\frac{dN^{11}}{dt} = -N^{11} \left(\sigma_a^{11} \Phi + \lambda^{11} \right) + N^{10} \left(\sigma_a^{10} - \sigma_f^{10} \right) \Phi \quad (8a)$$

$$N_{t+\Delta t}^{11} = \frac{N_t^{11} \left[2 - \left(\lambda^{11} + \sigma_a^{11} \Phi \right) \Delta t \right] + \left(N_{t+\Delta t}^{10} + N_t^{10} \right) \left(\sigma_a^{10} - \sigma_f^{10} \right) \Phi \Delta t}{2 + \left(\lambda^{11} + \sigma_a^{11} \Phi \right) \Delta t} \quad (8b)$$

U^{236} (VID = 6) or Pu^{242} (VID = 12)

$$\frac{dN^6}{dt} = -N^6 \sigma_a^6 \Phi + N^5 \left(\sigma_a^5 - \sigma_f^5 \right) \Phi \quad (9a)$$

$$N_{t+\Delta t}^6 = \frac{N_t^6 \left(2 - \sigma_a^6 \Phi \Delta t \right) + \left(N_{t+\Delta t}^5 + N_t^5 \right) \left(\sigma_a^5 - \sigma_f^5 \right) \Phi \Delta t}{2 + \sigma_a^6 \Phi \Delta t} \quad (9b)$$

Poisons: Explicit fission products (Xe^{135} , I^{135} , Sm^{149} , and Pm^{149}). - Each of these explicit fission products is treated under a separate heading. The analytic solutions enable the rapidly saturating fission products to be followed within a single time step. For each of the four products an equilibrium concentration equation is given, and in the case of Xe^{135} the buildup after shutdown is treated.

Xe^{135} : The equation governing the time behavior of the Xe^{135} nuclide concentration $X(t)$ is

$$\frac{dX(t)}{dt} = \lambda_I I(t) + \sum_{i=1}^{NF} \Gamma_i^X \sum_j^{NG} \Phi_j \Sigma_{f,j}^i - \lambda_X X(t) - \sigma_a^X \Phi X(t) - \xi_X X(t) \quad (10a)$$

where

$$\Sigma_{f,j}^i = \bar{N}^i \sigma_{f,j}^i = \frac{(N_{t+\Delta t}^i + N_t^i) \sigma_{f,j}^i}{2}$$

is the average macroscopic fission cross section for isotope i , energy group j ; ξ_X represents the removal rate for xenon other than by σ_a^X and λ_X ; and ξ_I is the corresponding quantity for iodine.

In the following derivations, in the interest of brevity, the time dependence will not always be shown and the summation over all energy groups will be implied. The Xe^{135} concentration at time $t + \Delta t$ is then

$$X_{t+\Delta t} = \frac{\lambda_I' [I_t (\lambda_I' + \sigma_a^I \Phi) - \Gamma^I]}{(\lambda_I' + \sigma_a^I \Phi)(\lambda_X^* + \sigma_a^X \Phi - \lambda_I' - \sigma_a^I \Phi)} \left\{ \exp \left[-(\lambda_I' + \sigma_a^I \Phi) \Delta t \right] - \exp \left[-(\lambda_X^* + \sigma_a^X \Phi) \Delta t \right] \right\} \\ + \frac{\Gamma^X (\lambda_I' + \sigma_a^I \Phi) + \Gamma^I \lambda_I'}{(\lambda_I' + \sigma_a^I \Phi)(\lambda_X^* + \sigma_a^X \Phi)} \left\{ 1 - \exp \left[-(\lambda_X^* + \sigma_a^X \Phi) \Delta t \right] \right\} + X_t \exp \left[-(\lambda_X^* + \sigma_a^X \Phi) \Delta t \right] \quad (10b)$$

in which $\lambda_X^* = \lambda_X + \xi_X$, $\lambda_I' = \lambda_I - \xi_I$, $\Gamma^K = \sum_i^{NF} \Gamma_i^K \sum_j^{NG} \Sigma_{f,j}^i$, and I_t and X_t represent concentrations of I^{135} and Xe^{135} at the start of the time interval. In the event I^{135} is not considered, equation (10b) is altered internally by letting $\lambda_I' = 0$ and $\Gamma^X = \Gamma^X + \Gamma^I$. The equilibrium or steady-state Xe^{135} concentration is given by

$$X_{ss} = \frac{\lambda_I' I_{ss} + \Gamma^X}{\lambda_X^* + \sigma_a^X \Phi} \quad (10c)$$

with Γ^X defined as in the preceding sentence and I_{ss} representing the I^{135} equilibrium concentration.

The equation for Xe^{135} concentration at a time Δt after shutdown may be obtained from equation (10b) by letting Φ , Γ^I , and Γ^X equal zero. Then, I_t and X_t become I_{sd} and X_{sd} , the respective concentrations at shutdown. The time after shutdown of maximum Xe^{135} concentration is

$$t_{\max} = \frac{1}{\lambda_X^* - \lambda_I'} \ln \left\{ \frac{\lambda_X^*}{\lambda_I'} \left[1 - \frac{(\lambda_X^* - \lambda_I') X_{sd}}{I_{sd}} \right] \right\} \quad (10d)$$

I^{135} : The isotope I^{135} does not have a decay precursor, thus its equation differs slightly from that of Xe^{135} (eq. (10a)). Another loss mechanism ξ_I (e.g., diffusion or expulsion) may be incorporated into λ_I , that is, $\lambda_I^* = \lambda_I + \xi_I$. The I^{135} nuclide concentration obeys the equation

$$\frac{dI(t)}{dt} = -\lambda_I^* I(t) - I(t) \sigma_a^I \Phi + \sum_{i=1}^{NF} \Gamma_i^I \sum_{j=1}^{NG} \Sigma_{fj}^i \Phi \quad (11a)$$

In finite difference form, equation (11a) becomes

$$I_{t+\Delta t} = I_t \exp \left[-(\lambda_I^* + \sigma_a^I \Phi) \Delta t \right] + \frac{\Gamma^I}{\lambda_I^* + \sigma_a^I \Phi} \left\{ 1 - \exp \left[-(\lambda_I^* + \sigma_a^I \Phi) \Delta t \right] \right\} \quad (11b)$$

The steady-state concentration is given by

$$I_{ss} = \frac{\Gamma^I}{\lambda_I^* + \sigma_a^I \Phi} \quad (11c)$$

Pm^{149} : The isotope Pm^{149} has the same creation and destruction mechanisms as I^{135} except that its effect on reactivity is small enough so that no additional loss mechanisms are considered. Thus the finite difference equation is

$$P_{t+\Delta t} = P_t \exp \left[-(\lambda_P + \sigma_a^P \Phi) \Delta t \right] + \left(\frac{\Gamma^P}{\lambda_P + \sigma_a^P \Phi} \right) \left\{ 1 - \exp \left[-(\lambda_P + \sigma_a^P \Phi) \Delta t \right] \right\} \quad (12a)$$

and the steady state concentration is given by

$$P_{ss} = \frac{\Gamma^P}{\lambda_p + \sigma_a^P \Phi} \quad (12b)$$

Sm¹⁴⁹: This isotope is similar to Xe¹³⁵ except that it is stable and no extra loss mechanisms are considered necessary. The equations for Sm¹⁴⁹ are

$$\frac{dS(t)}{dt} = \lambda_p P(t) + \sum_i^{NF} \Gamma_i^S \sum_{j=1}^{NG} \Sigma_{fj}^i \Phi - S(t) \sigma_a^S \Phi \quad (13a)$$

and

$$S_{t+\Delta t} = \frac{\lambda_p \left[P_t (\lambda_p + \sigma_a^P \Phi) - \Gamma^P \right]}{(\lambda_p + \sigma_a^P \Phi) (\sigma_a^S \Phi - \lambda_p - \sigma_a^P \Phi)} \left\{ \exp \left[-(\lambda_p + \sigma_a^P \Phi) \Delta t \right] - \exp \left(-\sigma_a^S \Phi \Delta t \right) \right\} \\ + \frac{\Gamma^S (\lambda_p + \sigma_a^P \Phi) + \Gamma^P \lambda_p}{(\lambda_p + \sigma_a^P \Phi) (\sigma_a^S \Phi)} P \left[1 - \exp \left(-\sigma_a^S \Phi \Delta t \right) \right] + S_t \exp \left(-\sigma_a^S \Phi \Delta t \right) \quad (13b)$$

In the event Pm¹⁴⁹ is not considered, equation (13b) is altered internally by letting $\lambda_p = 0$ and $\Gamma^S = \Gamma^S + \Gamma^P$. The steady state concentration is

$$S_{ss} = \frac{\lambda_p P_{ss} + \Gamma^S}{\sigma_a^S \Phi} \quad (13c)$$

Special materials: These materials, which include fission products and aggregates, obey the following depletion equation:

$$N_{t+\Delta t}^{SP} = \frac{\Gamma^{SP} + \frac{1}{2} (N_{t+\Delta t}^{SP-1} + N_t^{SP-1}) \sigma_a^{SP-1} \Phi \delta^{SP}}{\sigma_a^{SP} \Phi} \left[1 - \exp \left(-\sigma_a^{SP} \Phi \Delta t \right) \right] + N_t^{SP} \exp \left(-\sigma_a^{SP} \Phi \Delta t \right) \quad (14)$$

where $\delta^{SP} = 1$ if material SP is a capture product (e.g., (n, γ)) of the preceding material (SP - 1) and $\delta^{SP} = 0$ if it is not (see the section INPUT INSTRUCTIONS). For the fission product aggregates (VID = 13), $\delta^{SP} = 0$.

Burnable poisons: The concentrations of the nuclides considered as burnable poisons obey the simple analytic form

$$N_{t+\Delta t}^K = N_t^K \exp \left(- \sigma_a^K \Phi \Delta t \right) \quad (15)$$

Depletion cycle: These depletion equations may now be incorporated into a complete depletion program. Such a depletion program, coupled with the energy and space codes (e.g., GAM-GATHER and TDSN, refs. 6 to 8) enables a depletion cycle to be performed. A typical depletion cycle proceeds according to the flow diagram in figure 1.

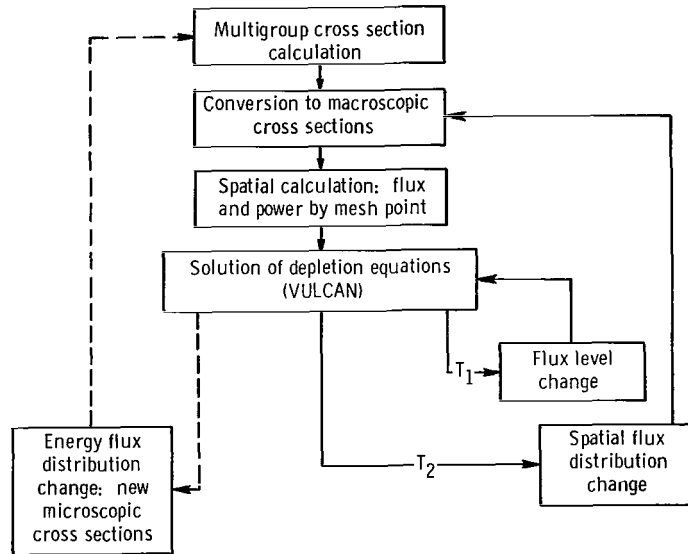


Figure 1. - VULCAN flow diagram.

Before entering the depletion program, two time intervals must be chosen. The shortest time interval T_1 will be that between renormalizations of the flux level (within VULCAN) to a given power. The other time interval T_2 will be that between recalculations to the spatial flux distribution. VULCAN will treat the T_2/T_1 normalizations sequentially as a single problem.

SAMPLE PROBLEM

This sample problem serves as a check on the solution of the depletion equations insofar as the original solution is concerned and as a device for displaying certain options of the program. To illustrate the flexibility of VULCAN, a problem previously solved by the depletion program CANDLE (ref. 2) but found in reference 9, was chosen as the sample problem. The problem has 2 energy groups and 14 mesh intervals comprising three regions in a slab geometry, and the depletion is over a single time step. Although the input to VULCAN illustrates the use of several options, only the output pertinent to

the sample problem in reference 9 is given. The input information (e. g. , cross sections and fluxes) listed in reference 9 was supplied to VULCAN. The first flux normalization factor in the VULCAN output (0.999738 E-09) indicates the degree to which this input was duplicated. For exact duplication of the variables controlling the power, this factor should be 1.0 E-09, with the exponent indicating the transition of flux level as used in the VULCAN input to neutrons per barn per second. The optional cross section input format used illustrates that particular option. Subroutine TABLE, as listed, contains current data; however, the following parameters assumed these values in the sample problem: THERNU(5) = 2.46, THERNU(9) = 2.88, THERNU(11) = 3.00, FIWATT = 3.15×10^{10} , DECAY(14) = 2.11×10^{-5} , DECAY(16) = 2.88×10^{-5} , and DECAY(17) = 3.85×10^{-6} .

A comparative list of atom densities at the end of the depletion interval is given in table I. In reference 9, however, only region-averaged atom densities were printed out. Although VULCAN does not normally yield atom densities averaged over the original regions, an appropriate value of the averaging criterion ϵ (see the section Subroutine OUTPUT) will yield these.

TABLE I. - COMPARATIVE ATOM DENSITIES

Nuclide	Material 1		Material 2	
	CANDLE (ref. 9)	VULCAN	CANDLE (ref. 9)	VULCAN
U ²³⁵	0.138761-3	0.138763-3	0.485625-4	0.485632-4
U ²³⁶	.500194-6	.500687-6	.937653-7	-----
Pm ¹⁴⁹	.261273-7	.261213-7	.563627-8	.561020-8
I ¹³⁵	.297741-7	.297672-7	.642296-8	.639333-8
Sm ¹⁴⁹	.505137-8	.505096-8	.136152-8	.135575-8
Xe ¹³⁵	.122126-8	.122129-8	.451111-9	.449453-9
FP2	.258698-5	.258238-5	.558072-6	.554921-6
U ²³⁸	-----	-----	.677028-2	.677051-2
Pu ²³⁹	-----	-----	.655909-6	.428466-6
Pu ²⁴⁰	-----	-----	.349750-8	.225741-8
Pu ²⁴¹	-----	-----	.227263-10	.145501-10

The discrepancy in Pu²³⁹, Pu²⁴⁰, and Pu²⁴¹ may be attributed primarily to the absence of a resonance correction to the smooth cross sections of U²³⁸ and Pu²³⁹; VULCAN assumes the cross sections directly reflect any resonance structure.

Input

1	VULCAN TEST	PROBLEM	FROM WAPD-TM-95	(WAPD-TM-221)	(CANDLE)	REF 9
2	1	18	3	2	3	22
6	14	2	0			
1	2	4	5	2	2	
1	2	1	0	0	1	
6495.33	1.0	1.0	50.0			
.01	.1	.5				
1	23	HYD				
2	24	OXY				
3	5	1U-235				
4	7	1U-238				
5	6	U-236				
6	17	PM149				
7	16	I-135				
8	15	SM149				
9	14	XE135				
10	13	FP2				
11	21	2ND25				
12	19	B10 1				
13	20	B10 2				
14	9	1PU239				
15	10	PU240				
16	11	1PU241				
17	18	SPMAT				
18	22	15 XFIS2				
1	2.54	1				
1	7.62	1				
1	12.7	2				
4	35.9	2				
1	38.1	2				
6	68.1	3				
13	1.0	1.0	1.0	1.0		
14	.003	.003	.003	.003		
15	.0	.0	.0	.0		
16	.06	.06	.06	.06		
17	.014	.014	.014	.014		
1.0						
1.0						
1.0						
1.0						
1.0						
1.0						
(36X,2F12.8/)						
HYDROGEN			.00444			
			.2151			
OXYGEN			.014			
U235			11.9416	8.8224		
			418.2	872.32		
U-238			.36457	.12498		
			1.79	0.		
U-236			.0	.0		
			3.9	.0		
PM-149			0.			
			0.			
I-135			0.			
			0.			

SM-149									
				0.					
				50000.					
XE-135				0.					
				2700000.					
FP2				0.					
				65.					
2ND25				12.1801		9.03777			
				418.2		872.32			
B10 1				60.					
				2470.					
B10 2				60.					
				2470.					
PU239				0.		0.			
				987.		1874.9			
PU240				0.		0.			
				373.		0.			
PU241				0.		0.			
				1080.		2415.			
SPMAT				60.					
				2470.					
XFIS2				11.9416					
				418.2					
	7	1	2	3	12	13	17		
.02738	18	.01369	.00014185	.001	.001	.001	.00014185		
.023643	4	.025464	.006771	.00004915	11				
.051714	2	.025857							
1.00431		.922701	.735115	.528068	.369309	.252466	.16132		
.103651		.0646481	.0314333	.0152257	.00725616	.0032126	.00090724		
.268374		.263683	.246155	.202483	.147527	.104308	.0801488		
.0830485		.096636	.085596	.0556303	.0315068	.0154594	.00458762		
1.25	2	1.25							
	3								
1.0	5	1.0	19	20					
.99		.80							
.99		.80							
.02738	5	.01369	.00014185	.001	.001		13		
.023643	4	.025464	.006771	.00004915	11				
.051714	2	.025857							
	3								
1.0	5	1.0	19	20					
.99		.80							
.99		.80							

Output

* MEMORY MAP *

SYSTEM 00000 THRU 02717
 FILE BLOCK ORIGIN 02720
 FILES 1. UNIT06 (NO BUFF POOL ATTACHED)
 2. UNIT05 (NO BUFF POOL ATTACHED)
 PRE-EXECUTION INITIALIZATION 02750
 CALL ON OBJECT PROGRAM 02757
 OBJECT PROGRAM 02764 THRU 76056

DECK	ORIGIN	CONTROL SECTIONS	(/NAME/=NON 0 LENGTH, (LOC)=DELETED, *=NOT REFERENCED)
1. VULCAN	02764	/SET1 / (02765)	/SET2 / (23615) /SET3 / (37445) /SET4 / (42235) /SET5 / (42603)
		/SET6 / (43461)	/SET7 / (43471) /SET8 / (43477) /SET9 / (43507) /SET10 / (43513)
		/SET11 / (43521) 51430 *
2. TTABLE	51454	/SET1 / (02765)	/SET2 / (23615) /SET3 / (37445) /SET4 / (42235) /SET5 / (42603)
		TABLE 51736	
3. TXENON	51764	/SET1 / (02765)	/SET2 / (23615) /SET3 / (37445) /SET4 / (42235) /SET5 / (42603)
		/SET6 / (43461)	/SET7 / (43471) /SET8 / (43477) /SET9 / (43507) /SET10 / (43513)
4. POUT	52525	/SET1 / (02765)	/SET2 / (23615) /SET3 / (37445) /SET4 / (42235) /SET5 / (42603)
		/SET6 / (43461)	/SET7 / (43471) /SET8 / (43477) /SET9 / (43507) /SET10 / (43513)
5. FADE	54623	/SET1 / (02765)	/SET2 / (23615) /SET3 / (37445) /SET4 / (42235) /SET5 / (42603)
		/SET6 / (43461)	/SET7 / (43471) /SET8 / (43477) /SET9 / (43507) /SET10 / (43513)
		FISEQ1 56026	FISEQ2 56031
6. ABFLUX	56034	/SET1 / (02765)	/SET2 / (23615) /SET3 / (37445) /SET4 / (42235) /SET5 / (42603)
		/SET6 / (43461)	/SET7 / (43471) /SET8 / (43477) /SET9 / (43507) /SET10 / (43513)
7. SELSH	56502	/SET1 / (02765)	/SET2 / (23615) /SET3 / (37445) /SET4 / (42235) /SET5 / (42603)
		/SET6 / (43461)	/SET7 / (43471) /SET8 / (43477) /SET9 / (43507) /SET10 / (43513)
8. BURNT	56764	/SET1 / (02765)	/SET2 / (23615) /SET3 / (37445) /SET4 / (42235) /SET5 / (42603)
		/SET6 / (43461)	/SET7 / (43471) /SET8 / (43477) /SET9 / (43507) /SET10 / (43513)
		/SET11 / (43521)	BURNUP 65766
9. LXCON	66006	.LXSTR 66006	.LXSTP 66011 .LXOUT 66033 * .LXERR 66042 * .LXCAL 66045
		.LXRTN 66045 *	.IBEXIT 66045 .DBCLS 66225 * .LXARG 66314 * .LO 66323 *
		.LXSE 66331 *	.LFBL 66332 * .LUNB 66333 * .DFOUT 66334
10. LXSL	66340	.LXSEL 66340	.LXSL1 66341 .LXTST 66344 .LXOVL 66404 .LXMOD 66446
		.LXINJ 66472	.LXD15 66475 * .LXFLG 66476 * .LTCH 66477
11. FPTRP	66505	.FPPT. 66505 *	.FXEM. 66507 * .FXARG 67103 * .EXIT. 67105 * .FMCRT 67052 *
		.SGOTD 67065 *	.FXOUT 67071 * .SYSONE 67140 .NOP 67141 .EXIT 67105 *
		.JVCCHK 67112 *	.OVFLOW 67113 * .SYSONE 67140 .NOP 67141 .EXIT 67105 *
12. FRAS.	67171	E.1 67171	E.2 57177 E.3 67173 E.4 67174
13. XCC.	67175	CC.1 67175	CC.2 67176 CC.3 67177 CC.4 67200
14. FCNV	67201	.FCNV. 67201	.DDPRE 57211 * .FCNV. 67217 .ENDFS 67230 .CNVSW 67232
		.FJX1 67236	.FJX2 67237 .DBC 67241 .DBCL14 67304 * .STOP4 67307 *
		.DBC20 67323	.DBC10 67337 .DBC99 67352 * .DDSW 67375 * .FJXSW 67404
		.JXPSE 67420 *	.DDZET 57525 * .IC10 67566 * .STOPJ 67603 * .FCOUT 67613 *
		.FCARG 67621 *	.ALCOD 67623 * .ANPT 67634 * .GNPT 67651 .LNTPT 67725
		.ADUT 67764	.LOUT 67776 .GOUT 70025 .TGOUT 70044 *
		.FLT 70144	.FXFL1 70301 .FXD 70305 .FXFL2 70311 .FXFL3 70315
		.INTG 70321	.TOPAC 70337 .WIOTH 70343 .FPACK 70350 .TEST 70351
		.JJSF 70402 *	.KOUNT 70403 .LIST 70406 .DONE 70415 .GUTBF 70551
		.CHAR 71003	.DEXP 71012 * .TEN 71016 * .FBD8F 71020 .DATUM 71026 *
		.WORD 71041	.MOD 71042 .PEX 71043 .FXP 71044 .DIG 71045
15. FIDH	71060	.FIDH. 71060	.DDIO 71105 * .DDLET 71243 * .DCPT1 71446 * .FXPT 71644 *
		.FFIL. 71675	.DDFIN 71713 * .FRTN. 71722 .DDRTN 71726 *
16. FIOS.	72030	.FIOS. 72030	.FCLS 72052 * .FIOC 72077 .FSEL 72120 .F93K 72137 *
		.FICK 72141	.IOFF 72225 * .XEM 72233 * .FCHK 72250 *
17. FRDD.	72353	.FRDD. 72353	
18. FRWD.	72400	.FRWD. 72400	
19. FBCC.	72426	.FBCC. 72426	
20. UNIT06	72471	.UN06 72471	.FRCM 72430 .FRCB 72441
21. UN05	72472	.UN05. 72472	
22. UN06	72473	.UN06. 72473	
23. FPUN	72474	.FPUN. 72474	
24. FLJG	72531	.ALOG10 72531 *	.ALOG 72532
25. FXPF	72706	EXP 72706	
26. IOE.	73011		
27. IOE56	73023		
28. RWDOE	73031		
29. BCREA	73051	BCREA 73051	BCREAD (73051)
30. BCRWD	73131	.BCRJ 73131	.BCWD 73133 * .BRDB 73145
31. FXP2	73203	.XP2. 73203	
32. PISTUG	73276	.JDLQ / 73277	.PISTUG (73276)
33. PLOTXY	74102	.JDLQ / (73277)	.PLOTXY (74102)
34. KHAR	75761	.KHAR (75761)	
35. LOGE.	76033	.LOGE 76033	
36. TLEX.	76041	.TLEX 76041	
37. XEXP.	76045	.NGDEF 76045	.XEXP 76051 * .ZUDEF 76054

UNUSED CORE 76057 THRU 77777

BEGIN EXECUTION.

VULCAN TEST PROBLEM FROM WAPD-TM-95 (WAPD-TM-221) (CANDLE) REF 9

NG 2 NFG 1 NISOT 18 NMAT 3 NTINC 2 NFRAC 3 KCELL 22

NZQNR 6 NRM 14 MCROUT 2 NUF 0

NSPMAT 1 NBPOI 2 NFIS 4 NYDNUC 5 NONDPL 2 NFIRPT 2

KGED 1 KFLUX 2 KSSF 1 KHAIN 0 KFAST 0 KFORM 1

POWER IN FIRST SECOND TIME XENON IODINE
WATTS DISTANCE DISTANCE INCREMENT REMOVAL REMOVAL
.64953E 04 1.00000 1.00000 50.30 -0. -0. DIFFER
-0.

SEQUENTIAL ID	VULCAN ID	NRPT	ISOTOPE LABEL
1	23	-0	HVO
2	24	-0	OXY
3	5 Y	-0	U-235
4	7 Y	-0	U-238
5	6	-0	U-236
6	17	-0	PM149
7	16	-0	I-135
8	15	-0	SM149
9	14	-0	XE135
10	13	-0	FP2
11	21	15	2ND25
12	19	-0	B10 1
13	20	-0	B10 2
14	9 Y	-0	PU239
15	10	-0	PU240
16	11 Y	-0	PU241
17	18	-0	SPMAT
18	22	15	XFIS2

NMIR(I)	RAR(I)	MIDRZ(I)-MAP
1	2.5400	1
1	7.6200	1
1	12.7000	2
4	35.9000	2 2 2 2
1	38.1000	2
6	68.1000	3 3 3 3 3

MESH VOLUMES

0.254000E 01 0.508000E 01 0.508000E 01 0.580000E 01 0.580000E 01 0.580000E 01 0.580000E 01 0.580000E 01 0.220000E 01
0.500000E 01 0.500000E 01 0.500000E 01 0.500000E 01 0.500000E 01 0.500000E 01 0.681000E 02

ZONE VOLUMES

0.254000E 01 0.508000E 01 0.508000E 01 0.232000E 02 0.220000E 01 0.300000E 02

FISSION PRODUCT	YIELD FROM 5 (U-235)	YIELD FROM 7 (U-238)	YIELD FROM 9 (PU239)	YIELD FROM 11 (PU241)	YIELD FROM (YIELD	YIELD	YIELD	YIELD	YIELD
17 (PM149)	0.01400	0.01400	0.01400	0.01400						
16 (I-135)	0.06000	0.06000	0.06000	0.06000						
15 (SM149)	0.	0.	0.	0.						
14 (XE135)	0.00300	0.00300	0.00300	0.00300						
13 (FP2)	1.00000	1.00000	1.00000	1.00000						
18 (SPMAT)	0.	0.	0.	0.						

GROUP	NU FOR VID 1	NU FOR VID 2	NU FOR VID 3	NU FOR VID 4	NU FOR VID 5	NU FOR VID 6	NU FOR VID 7	NU FOR VID 8	NU FOR VID 9	NU FOR VID 10	NU FOR VID 11	NU FOR VID 12
1	0.	0.	0.	0.	1.000	1.000	1.000	0.	1.000	1.000	1.000	0.
T	2.600	0.	2.503	0.	2.460	0.	2.600	0.	2.880	0.	3.000	0.

SEQUENTIAL ID = 1 VULCAN ID = 23

ABSORPTION BY GROUP (HIGH TO LOW)
0.444000E-02 0.215100E 00

SEQUENTIAL ID = 2 VULCAN ID = 24

ABSORPTION BY GROUP (HIGH TO LOW)
0.140000E-01 0.

SEQUENTIAL ID = 3 VULCAN ID = 5

ABSORPTION BY GROUP (HIGH TO LOW)
0.119416E 02 0.418200E 03
NU*FISSION BY GROUP (HIGH TO LOW)
0.882240E 01 0.872320E 03
FISSION BY GROUP (HIGH TO LOW)
0.882240E 01 0.354602E 03

SEQUENTIAL ID = 4 VULCAN ID = 7

ABSORPTION BY GROUP (HIGH TO LOW)
0.364570E 00 0.179000E 01
NU*FISSION BY GROUP (HIGH TO LOW)
0.124980E 00 0.
FISSION BY GROUP (HIGH TO LOW)
0.124980E 00 0.

SEQUENTIAL ID = 5 VULCAN ID = 6

ABSORPTION BY GROUP (HIGH TO LOW)
0. 0.390000E 01
NU*FISSION BY GROUP (HIGH TO LOW)
0. 0.
FISSION BY GROUP (HIGH TO LOW)
0. 0.

SEQUENTIAL ID = 6 VULCAN ID = 17

ABSORPTION BY GROUP (HIGH TO LOW)
0. 0.

SEQUENTIAL ID = 7 VULCAN ID = 16

ABSORPTION BY GROUP (HIGH TO LOW)
0. 0.

SEQUENTIAL ID = 8 VULCAN ID = 15

ABSORPTION BY GROUP (HIGH TO LOW)
0. 0.500000E 05

SEQUENTIAL ID = 9 VULCAN ID = 14

ABSORPTION BY GROUP (HIGH TO LOW)
0. 0.270000E 07

SEQUENTIAL ID = 10 VULCAN ID = 13

ABSORPTION BY GROUP (HIGH TO LOW)
0. 0.650000E 02

SEQUENTIAL ID = 11 VULCAN ID = 21

ABSORPTION BY GROUP (HIGH TO LOW)
0.121801E 02 0.418200E 03
NU*FISSION BY GROUP (HIGH TO LOW)
0.903777E 01 0.872320E 03
FISSION BY GROUP (HIGH TO LOW)
0.903777E 01 0.354602E 03

SEQUENTIAL ID = 12 VULCAN ID = 19

ABSORPTION BY GROUP (HIGH TO LOW)
0.600000E 02 0.247000E 04

SEQUENTIAL ID = 13 VULCAN ID = 20

ABSORPTION BY GROUP (HIGH TO LOW)
0.600000E 02 0.247000E 04

SEQUENTIAL ID = 14 VULCAN ID = 9

ABSORPTION BY GROUP (HIGH TO LOW)
0. 0.987000E 03
NU*FISSION BY GROUP (HIGH TO LOW)
0. 0.187490E 04
FISSION BY GROUP (HIGH TO LOW)
0. 0.651007E 03

SEQUENTIAL ID = 15 VULCAN ID = 10

ABSORPTION BY GROUP (HIGH TO LOW)
 0. 0.373000E 03
 NU*FISSION BY GROUP (HIGH TO LOW)
 0. 0.
 FISSION BY GROUP (HIGH TO LOW)
 0. 0.

SEQUENTIAL ID = 16 VULCAN ID = 11

ABSORPTION BY GROUP (HIGH TO LOW)
 0. 0.108000E 04
 NU*FISSION BY GROUP (HIGH TO LOW)
 0. 0.241500E 04
 FISSION BY GROUP (HIGH TO LOW)
 0. 0.805000E 03

SEQUENTIAL ID = 17 VULCAN ID = 18

ABSORPTION BY GROUP (HIGH TO LOW)
 0.600000E 02 0.247000E 04

SEQUENTIAL ID = 18 VULCAN ID = 22

ABSORPTION BY GROUP (HIGH TO LOW)
 0.119416E 02 0.418200E 03
 NU*FISSION BY GROUP (HIGH TO LOW)
 -0. -0.
 FISSION BY GROUP (HIGH TO LOW)
 -0. -0.

VULCAN ID	ISOTOPE LABEL	MATERIAL 1	MATERIAL 2	MATERIAL 3	MATERIAL 0	MATERIAL 0	MATERIAL 0	MATERIAL 0
23	HYD	0.27380E-01	0.23643E-01	0.51714E-01				
24	OXY	0.13690E-01	0.25464E-01	0.25857E-01				
5	U-235	0.14185E-03	0.	0.				
7	U-238	0.	0.67710E-02	0.				
6	U-236	0.	0.	0.				
17	PM149	0.	0.	0.				
16	I-135	0.	0.	0.				
15	SM149	0.	0.	0.				
14	XE135	0.	0.	0.				
13	FP2	0.	0.	0.				
21	2ND25	0.	0.49150E-04	0.				
19	B1C 1	0.10000E-02	0.	0.				
20	B1O 2	0.10000E-02	0.	0.				
9	PU239	0.	0.	0.				
10	PU240	0.	0.	0.				
11	PU241	0.	0.	0.				
18	SPMAT	0.10000E-02	0.	0.				
22	XFIS2	0.14185E-03	0.	0.				

FLUXES BY GROUP AND MESH POINT
FOR THE BASIC CELL

GROUP	1	2	3	4	5	6	7	8	9	10
0.100431E-01	0.922701E-00	0.735115E-00	0.528068E-00	0.369309E-00	0.252466E-00	0.161320E-00	0.103651E-00			
0.646481E-01	0.314333E-01	0.152257E-01	0.725615E-02	0.321260E-02	0.907240E-03					
0.268374E-00	0.263683E-00	0.246155E-00	0.202483E-00	0.147527E-00	0.104308E-00	0.801488E-01	0.830485E-01			
0.966360E-01	0.855960E-01	0.556303E-01	0.315063E-01	0.154594E-01	0.458762E-02					

FOR THE START OF TIME INTERVAL NUMBER 1 OF 2

FLUX NORMALIZATION FACTOR 0.999738E-09 CONVERTS TO ABSOLUTE FLUX IN NEUTRONS/BARN*SEC
FOR A POWER LEVEL OF 0.649533E 04 WATTS

CELL NUMBER 1 OF 2

THE POWER FOR EACH CORE REGION
0.125000E 01 0.125000E 01

THESE ARE THE RADIAL POWER FACTORS
0.100000E 01 0.100000E 01

THE POWER FACTOR FOR THIS CELL IS 1.0000000
FLUXES BY GROUP AND MESH POINT
FOR CELL NUMBER 1

GROUP	1	2	3	4	5	6	7	8	9	10
0.100405E-08	0.922459E-09	0.734922E-09	0.527930E-09	0.369212E-09	0.252400E-09	0.161278E-09	0.103624E-09			
0.646311E-10	0.314281E-10	0.152217E-10	0.725425E-11	0.321176E-11	0.907002E-12					
0.268304E-09	0.263614E-09	0.246090E-09	0.202430E-09	0.147488E-09	0.104281E-09	0.801278E-10	0.830267E-10			
0.966107E-10	0.855736E-10	0.556157E-10	0.314985E-10	0.154553E-10	0.458642E-11					

SELF-SHIELDING FACTORS USED ARE (HIGH TO LOW ENERGY)

VULCAN ID = 5	1.000000	1.000000
VULCAN ID = 19	0.990000	0.800000
VULCAN ID = 20	0.990000	0.800000

** ** * ** * ** * ** * MESH INTERVAL NUMBER 1 ** ** * ** * ** * ** * ** *

FISSION PRODUCT CONCENTRATIONS AT VARIOUS FRACTIONS OF THE TIME STEP

FRACTION	XE135	SM149	I 135	PM149
0.01	0.702148E-10	0.126074E-11	0.153549E-08	0.366376E-09
0.10	0.505588E-09	0.115001E-09	0.122953E-07	0.355198E-08
0.50	0.114142E-08	0.195720E-08	0.291185E-07	0.155357E-07
EQUILIBRIUM	0.123286E-08	0.152252E-07	0.303941E-07	0.530516E-07

TIME AFTER SHUTDOWN UNTIL MAXIMUM XENON CONCENTRATION IS REACHED = 10.83 HOURS
THE XENON-135 CONCENTRATION (PEAK) AT THIS TIME IS 0.134919E-07

ISOTOPE DESCR.	N(T)	N(T+1)	N(T+1)/N(T)	POWER FRACTION	POISON FACTOR
HYD - 1	0.273800E-01	0.273800E-01	0.100000E 01	0.	0.494032E-01
OXY - 2	0.136900E-01	0.136900E-01	0.100000E 01	0.	0.558504E-02
U-235- 3	0.141850E-03	0.138714E-03	0.977892E 00	0.100000E 01	0.499952E 00
U-238- 4	0.	0.	0.	0.	0.
U-236- 5	0.	0.509904E-06	0.	0.	0.154853E-04
PM149- 6	0.	0.265219E-07	0.	0.	0.
I-135- 7	0.	0.302238E-07	0.	0.	0.
SM149- 8	0.	0.509474E-08	0.	0.	0.198363E-02
XE135- 9	0.	0.122601E-08	0.	0.	0.257766E-01
FP2 -10	0.	0.262193E-05	0.	0.	0.132710E-02
2ND25-11	0.	0.	0.	0.	0.
B10 1-12	0.100000E-02	0.899276E-03	0.899276E 00	0.	0.153938E 02
B10 2-13	0.100000E-02	0.899276E-03	0.899276E 00	0.	0.153938E 02
PU239-14	0.	0.	0.	0.	0.
PU240-15	0.	0.	0.	0.	0.
PU241-16	0.	0.	0.	0.	0.
SPMAT-17	0.100000E-02	0.877980E-03	0.877980E 00	0.	0.184219E 02
XFI52-18	0.141850E-03	0.138714E-03	0.977892E 00	-0.	0.499952E 00

** ** * ** * ** * ** * MESH INTERVAL NUMBER 2 ** ** * ** * ** * ** * ** *

FISSION PRODUCT CONCENTRATIONS AT VARIOUS FRACTIONS OF THE TIME STEP

FRACTION	XF135	SM149	I 135	PM149
0.01	0.691652E-10	0.123237E-11	0.150070E-08	0.358076E-09
0.10	0.502168E-09	0.112548E-09	0.120167E-07	0.347150E-08
0.50	0.113477E-08	0.192420E-08	0.274915E-07	0.151837E-07
EQUILIBRIUM	0.122574E-08	0.151450E-07	0.297055E-07	0.518496E-07

TIME AFTER SHUTDOWN UNTIL MAXIMUM XENON CONCENTRATION IS REACHED = 10.83 HOURS
THE XENON-135 CONCENTRATION (PEAK) AT THIS TIME IS 0.131954E-07

ISOTOPE DESCR.	N(T)	N(T+1)	N(T+1)/N(T)	POWER FRACTION	POISON FACTOR
HYD - 1	0.273800E-01	0.273800E-01	0.100000E 01	0.	0.494573E-01
OXY - 2	0.136900E-01	0.136900E-01	0.100000E 01	0.	0.525265E-02
U-235- 3	0.141850E-03	0.138787E-03	0.978409E 00	0.100000E 01	0.499952E 00
U-238- 4	0.	0.	0.	0.	0.
U-236- 5	0.	0.496078E-06	0.	0.	0.151524E-04
PM149- 6	0.	0.259210E-07	0.	0.	0.
I-135- 7	0.	0.295390E-07	0.	0.	0.
SM149- 8	0.	0.502907E-08	0.	0.	0.196936E-02
XE135- 9	0.	0.121893E-08	0.	0.	0.257757E-01
FP2 -10	0.	0.256260E-05	0.	0.	0.130455E-02
2ND25-11	0.	0.	0.	0.	0.
B10 1-12	0.100000E-02	0.901563E-03	0.901563E 00	0.	0.154201E 02
B10 2-13	0.100000E-02	0.901563E-03	0.901563E 00	0.	0.154201E 02
PU239-14	0.	0.	0.	0.	0.
PU240-15	0.	0.	0.	0.	0.
PU241-16	0.	0.	0.	0.	0.
SPMAT-17	0.100000E-02	0.880588E-03	0.880588E 00	0.	0.184828E 02
XFI52-18	0.141850E-03	0.138787E-03	0.978409E 00	-0.	0.499952E 00

*** ** MESH INTERVAL NUMBER 3 *** **

FISSION PRODUCT CONCENTRATIONS AT VARIOUS FRACTIONS OF THE TIME STEP

FRACTION	XE135	SM149	I 135	PM149
0.01	0.262346E-10	0.454001E-12	0.552582E-09	0.131849E-09
0.10	0.196862E-09	0.416535E-10	0.442475E-08	0.127826E-08
0.50	0.446576E-09	0.724495E-09	0.101191E-07	0.559089E-08
EQUILIBRIUM	0.482488E-09	0.597372E-08	0.109381E-07	0.190919E-07

TIME AFTER SHUTDOWN UNTIL MAXIMUM XENON CONCENTRATION IS REACHED = 10.80 HOURS
THE XENON-135 CONCENTRATION (PEAK) AT THIS TIME IS 0.487247E-08

ISOTOPE DESCR.	N(T)	N(T+1)	N(T+1)/N(T)	POWER FRACTION	POISON FACTOR
HYD - 1	0.236430E-01	0.236430E-01	0.100000E 01	0.	0.128201E 00
OXY - 2	0.254640E-01	0.254640E-01	0.100000E 01	0.	0.252797E-01
U-235- 3	0.	0.	0.	0.	0.
U-238- 4	0.677100E-02	0.677014E-02	0.999872E 00	0.118150E 00	0.462777E 00
U-236- 5	0.	0.	0.	0.	0.
PM149- 6	0.	0.954454E-08	0.	0.	0.
I-135- 7	0.	0.108767E-07	0.	0.	0.
SM149- 8	0.	0.192306E-08	0.	0.	0.228314E-02
XE135- 9	0.	0.479799E-09	0.	0.	0.307605E-01
FP2 -10	0.	0.943688E-06	0.	0.	0.145650E-02
2ND25-11	0.491500E-04	0.481702E-04	0.980065E 00	0.859464F 00	0.519940E 00
B10 1-12	0.	0.	0.	0.	0.
B10 2-13	0.	0.	0.	0.	0.
PU239-14	0.	0.735355E-06	0.	0.223838E-01	0.172339E-01
PU240-15	0.	0.542739E-08	0.	0.	0.480696E-04
PU241-16	0.	0.437833E-10	0.	0.164799E-05	0.112280E-05
SPMAT-17	0.	0.	0.	0.	0.
XFIS2-18	0.	0.	0.	-0.	0.

*** ** MESH INTERVAL NUMBER 4 *** **

FISSION PRODUCT CONCENTRATIONS AT VARIOUS FRACTIONS OF THE TIME STEP

FRACTION	XE135	SM149	I 135	PM149
0.01	0.227396E-10	0.365149E-12	0.443964E-09	0.105908E-09
0.10	0.188422E-09	0.338872E-10	0.355419E-08	0.102677E-08
0.50	0.432886E-09	0.615993E-09	0.812822E-08	0.449090E-08
EQUILIBRIUM	0.468043E-09	0.583334E-08	0.878603E-08	0.153356E-07

TIME AFTER SHUTDOWN UNTIL MAXIMUM XENON CONCENTRATION IS REACHED = 10.71 HOURS
THE XENON-135 CONCENTRATION (PEAK) AT THIS TIME IS 0.394938E-08

ISOTOPE DESCR.	N(T)	N(T+1)	N(T+1)/N(T)	POWER FRACTION	POISON FACTOR
HYD - 1	0.236430E-01	0.236430E-01	0.100000E 01	0.	0.131065E 00
OXY - 2	0.254640E-01	0.254640E-01	0.100000E 01	0.	0.227367E-01
U-235- 3	0.	0.	0.	0.	0.
U-238- 4	0.677100E-02	0.677032E-02	0.999900E 00	0.105724E 00	0.453751E 00
U-236- 5	0.	0.	0.	0.	0.
PM149- 6	0.	0.766668E-08	0.	0.	0.
I-135- 7	0.	0.873678E-08	0.	0.	0.
SM149- 8	0.	0.170460E-08	0.	0.	0.208432E-02
XE135- 9	0.	0.465411E-09	0.	0.	0.307306E-01
FP2 -10	0.	0.758215E-06	0.	0.	0.120525E-02
2ND25-11	0.491500E-04	0.483507E-04	0.983738E 00	0.876022E 00	0.532051E 00
B10 1-12	0.	0.	0.	0.	0.
B10 2-13	0.	0.	0.	0.	0.
PU239-14	0.	0.585230E-06	0.	0.182531E-01	0.141259E-01
PU240-15	0.	0.355822E-08	0.	0.	0.324573E-04
PU241-16	0.	0.237101E-10	0.	0.914434E-06	0.626222E-06
SPMAT-17	0.	0.	0.	0.	0.
XFIS2-18	0.	0.	0.	-0.	0.

** ** * MESH INTERVAL NUMBER 5 ** ** *

FISSION PRODUCT CONCENTRATIONS AT VARIOUS FRACTIONS OF THE TIME STEP

FRACTION	XE135	SM149	I 135	PM149
0.01	0.181886E-10	0.264636E-12	0.321126E-09	0.766226E-10
0.10	0.179272E-09	0.249158E-10	0.257139E-08	0.742847E-09
0.50	0.423315E-09	0.480016E-09	0.588060E-08	0.324908E-08
EQUILIBRIUM	0.458413E-09	0.579243E-08	0.635652E-08	0.110950E-07

TIME AFTER SHUTDOWN UNTIL MAXIMUM XENON CONCENTRATION IS REACHED = 10.53 HOURS
THE XENON-135 CONCENTRATION (PEAK) AT THIS TIME IS 0.291075E-08

ISOTOPE DESCR.	N(T)	N(T+1)	N(T+1)/N(T)	POWER FRACTION	POISON FACTOR
HYD - 1	0.236430E-01	0.236430E-01	0.100000E 01	0.	0.132028E 00
OXY - 2	0.254640E-01	0.254640E-01	0.100000E 01	0.	0.220301E-01
U-235- 3	0.	0.	0.	0.	0.
U-238- 4	0.677100E-02	0.677051E-02	0.999928E 00	0.102256F 00	0.4517C3E 00
U-236- 5	0.	0.	0.	0.	0.
PM149- 6	0.	0.554669E-08	0.	0.	0.
I-135- 7	0.	0.632088E-08	0.	0.	0.173941E-02
SM149- 8	0.	0.140941E-08	0.	0.	0.303786E-01
XE135- 9	0.	0.455785E-09	0.	0.	0.880470E-03
FP2 -10	0.	0.54872 9E-06	0.	0.	0.537949E 00
2ND25-11	0.491500F-04	0.485680E-04	0.988159E 03	0.884419E 00	0.537949E 00
B10 1-12	0.	0.	0.	0.	0.
B10 2-13	0.	0.	0.	0.	0.
PU239-14	0.	0.424005E-06	0.	0.133249F-01	0.103307E-01
PU240-15	0.	0.188173E-08	0.	0.	0.173264E-04
PU241-16	0.	0.918375F-11	0.	0.356880E-06	0.244843E-06
SPMAT-17	0.	0.	0.	0.	0.
XF1S2-18	0.	0.	0.	-0.	0.

** ** * MESH INTERVAL NUMBER 6 ** ** *

FISSION PRODUCT CONCENTRATIONS AT VARIOUS FRACTIONS OF THE TIME STEP

FRACTION	XE135	SM149	I 135	PM149
0.01	0.138902E-10	0.186272E-12	0.225782E-09	0.538729E-10
0.10	0.166770E-09	0.177436E-10	0.180793E-08	0.522292E-09
0.50	0.411354E-09	0.358588E-09	0.413462E-08	0.228441E-08
EQUILIBRIUM	0.446543E-09	0.576008E-08	0.445923E-08	0.780084E-08

TIME AFTER SHUTDOWN UNTIL MAXIMUM XENON CONCENTRATION IS REACHED = 10.27 HOURS
THE XENON-135 CONCENTRATION (PEAK) AT THIS TIME IS 0.210290E-08

ISOTOPE DESCR.	N(T)	N(T+1)	N(T+1)/N(T)	POWER FRACTION	POISON FACTOR
HYD - 1	0.236430E-01	0.236430E-01	0.100000E 01	0.	0.132801E 00
OXY - 2	0.254640E-01	0.254640E-01	0.100000E 01	0.	0.214598E-01
U-235- 3	0.	0.	0.	0.	0.
U-238- 4	0.677100E-02	0.677066E-02	0.999950E 00	0.994652F-01	0.4500C6E 00
U-236- 5	0.	0.	0.	0.	0.
PM149- 6	0.	0.389985E-08	0.	0.	0.
I-135- 7	0.	0.444418E-08	0.	0.	0.
SM149- 8	0.	0.110963E-08	0.	0.	0.137985E-02
XE135- 9	0.	0.443908E-09	0.	0.	0.298086E-01
FP2 -10	0.	0.385904E-06	0.	0.	0.623847E-03
2ND25-11	0.491500E-04	0.487387E-04	0.991632E 03	0.891100F 00	0.542660E 00
B10 1-12	0.	0.	0.	0.	0.
B10 2-13	0.	0.	0.	0.	0.
PU239-14	0.	0.298431F-06	0.	0.943504E-02	0.732565E-02
PU240-15	0.	0.937784E-09	0.	0.	0.869955E-05
PU241-16	0.	0.32494 6E-11	0.	0.127035E-06	0.872817E-07
SPMAT-17	0.	0.	0.	0.	0.
XF1S2-18	0.	0.	0.	-0.	0.

** ** * MESH INTERVAL NUMBER 7 ** ** *

FISSION PRODUCT CONCENTRATIONS AT VARIOUS FRACTIONS OF THE TIME STEP

FRACTION	XE135	SM149	I 135	PM149
0.01	0.109043E-10	0.139640E-12	0.158999E-09	0.403241E-10
0.10	0.151570E-09	0.133770E-10	0.135324E-08	0.390937E-09
0.50	0.391463E-09	0.277898E-09	0.309478E-08	0.170989E-08
EQUILIBRIUM	0.426035E-09	0.561103E-08	0.334523E-08	0.583895E-08

TIME AFTER SHUTDOWN UNTIL MAXIMUM XENON CONCENTRATION IS REACHED = 10.02 HOURS
THE XENON-135 CONCENTRATION (PEAK) AT THIS TIME IS 0.161651E-08

ISOTOPE DESCR.	N(T)	N(T+1)	N(T+1)/N(T)	POWER FRACTION	POISON FACTOR
HYD - 1	0.236430E-01	0.236430E-01	0.100000E 01	0.	0.136074E 00
OKY - 2	0.254640E-01	0.254640E-01	0.100000E 01	0.	0.184331E-01
U-235- 3	0.	0.	0.	0.	0.
U-238- 4	0.677100E-02	0.677075E-02	0.999964E 00	0.849365E-01	0.438979E 00
U-236- 5	0.	0.	0.	0.	0.
PM149- 6	0.	0.291905E-08	0.	0.	0.
I-135- 7	0.	0.332648E-08	0.	0.	0.
SM149- 8	0.	0.887707E-09	0.	0.	0.114023E-02
XE135- 9	0.	0.423447E-09	0.	0.	0.293708E-01
FP2 -10	0.	0.288892E-06	0.	0.	0.482354E-03
2ND25-11	0.491500E-04	0.488372E-04	0.993635E 00	0.907911E 00	0.555429E 00
B10 1-12	0.	0.	0.	0.	0.
B10 2-13	0.	0.	0.	0.	0.
PU239-14	0.	0.220330E-06	0.	0.715295E-02	0.558656E-02
PU240-15	0.	0.532431E-09	0.	0.	0.510182E-05
PU241-16	0.	0.142090E-11	0.	0.570408E-07	0.394222E-07
SPMAT-17	0.	0.	0.	0.	0.
XFIS2-18	0.	0.	0.	-0.	0.

** ** * MESH INTERVAL NUMBER 8 ** ** *

FISSION PRODUCT CONCENTRATIONS AT VARIOUS FRACTIONS OF THE TIME STEP

FRACTION	XE135	SM149	I 135	PM149
0.01	0.106788E-10	0.137407E-12	0.165463E-09	0.397189E-10
0.10	0.145596E-09	0.131649E-10	0.133293E-08	0.385070E-09
0.50	0.373428E-09	0.272579E-09	0.304833E-08	0.168423E-08
EQUILIBRIUM	0.406250E-09	0.533385E-08	0.329503E-08	0.575133E-08

TIME AFTER SHUTDOWN UNTIL MAXIMUM XENON CONCENTRATION IS REACHED = 10.05 HOURS
THE XENON-135 CONCENTRATION (PEAK) AT THIS TIME IS 0.158600E-08

ISOTOPE DESCR.	N(T)	N(T+1)	N(T+1)/N(T)	POWER FRACTION	POISON FACTOR
HYD - 1	0.236430E-01	0.236430E-01	0.100000E 01	0.	0.142640E 00
OKY - 2	0.254640E-01	0.254640E-01	0.100000E 01	0.	0.121660E-01
U-235- 3	0.	0.	0.	0.	0.
U-238- 4	0.677100E-02	0.677077E-02	0.999966E 00	0.554120E-01	0.415632E 00
U-236- 5	0.	0.	0.	0.	0.
PM149- 6	0.	0.287524E-08	0.	0.	0.
I-135- 7	0.	0.327656E-08	0.	0.	0.
SM149- 8	0.	0.867318E-09	0.	0.	0.118577E-02
XE135- 9	0.	0.403793E-09	0.	0.	0.298108E-01
FP2 -10	0.	0.284552E-06	0.	0.	0.505739E-03
2ND25-11	0.491500E-04	0.488327E-04	0.993544E 00	0.937421E 00	0.578700E 00
B10 1-12	0.	0.	0.	0.	0.
B10 2-13	0.	0.	0.	0.	0.
PU239-14	0.	0.209840E-06	0.	0.716734E-02	0.566315E-02
PU240-15	0.	0.525376E-09	0.	0.	0.535835E-05
PU241-16	0.	0.145240E-11	0.	0.613429E-07	0.428905E-07
SPMAT-17	0.	0.	0.	0.	0.
XFIS2-18	0.	0.	0.	-0.	0.

*** ** ** ** ** ** MESH INTERVALS NUMBER 9 THRU 14 *** ** ** ** **

NO FISSIONABLE ISOTOPES OR CONSEQUENT FISSION PRODUCTS OCCUR IN MESH INTERVALS 9 THRU 14

ISOTOPE DESCR.	N(T)	N(T+1)	N(T+1)/N(T)	POWER FRACTION	POISON FACTOR
HYD - 1	0.517140E-01	0.517140E-01	0.100000F 01	0.	0.
OXY - 2	0.258570E-01	0.258570E-01	0.100000F 01	0.	0.
U-235- 3	0.	0.	0.	0.	0.
U-238- 4	0.	0.	0.	0.	0.
U-236- 5	0.	0.	0.	0.	0.
PM149- 6	0.	0.	0.	0.	0.
I-135- 7	0.	0.	0.	0.	0.
SM149- 8	0.	0.	0.	0.	0.
XE135- 9	0.	0.	0.	0.	0.
FP2 -10	0.	0.	0.	0.	0.
2ND25-11	0.	0.	0.	0.	0.
B10 1-12	0.	0.	0.	0.	0.
B10 2-13	0.	0.	0.	0.	0.
PU239-14	0.	0.	0.	0.	0.
PU240-15	0.	0.	0.	0.	0.
PU241-16	0.	0.	0.	0.	0.
SPMAT-17	0.	0.	0.	0.	0.
XFIS2-18	0.	0.	0.	0.	0.

ALL MESH INTERVALS HAVE BEEN TRAVERSED FOR TIME INTERVAL 1
THE FLUX WILL NOW BE RENORMALIZED FOR THE NEXT TIME INTERVAL

FOR THE START OF TIME INTERVAL NUMBER 2 OF 2

FLUX NORMALIZATION FACTOR 0.101036E 01 CONVERTS TO ABSOLUTE FLUX IN NEUTRONS/BARN*SEC
FOR A POWER LEVEL OF 0.649533E 34 WATTS

CELL NUMBER 1 OF 2

NEW ATOM DENSITIES BY MESH INTERVAL FOR EACH ISOTOPE AT THE END OF TIME INTERVAL 1

LAST ENTRY FOR EACH ISOTOPE IS ONE BARN-TH OF ITS TOTAL NUMBER OF ATOMS IN THE CONFIGURATION

VULCAN ID = 23							
0.273800E-01	0.273800E-01	0.236430E-01	0.236430E-01	0.236430E-01	0.236430E-01	0.236430E-01	0.236430E-01
0.517140E-01	0.517140E-01	0.517140E-01	0.517140E-01	0.517140E-01	0.517140E-01	0.248069E 01	0.236430E-01
VULCAN ID = 24							
0.136900E-01	0.136900E-01	0.254640E-01	0.254640E-01	0.254640E-01	0.254640E-01	0.254640E-01	0.254640E-01
0.258570E-01	0.258570E-01	0.258570E-01	0.258570E-01	0.258570E-01	0.258570E-01	0.165617E 01	0.254640E-01
VULCAN ID = 5							
0.138714E-03	0.138787E-03	0.	0.	0.	0.	0.	0.
0.	0.	0.	0.	0.	0.	0.105737E-02	0.
VULCAN ID = 7							
0.	0.	0.677014E-02	0.677032E-02	0.677051E-02	0.677066E-02	0.677075E-02	0.677077E-02
0.	0.	0.	0.	0.	0.	0.206365E 00	0.
VULCAN ID = 6							
0.509904E-06	0.466078E-06	0.	0.	0.	0.	0.	0.
0.	0.	0.	0.	0.	0.	0.381523E-05	0.
VULCAN ID = 17							
0.265219E-07	0.259210E-07	0.954454E-08	0.765668E-08	0.554669E-08	0.389985E-08	0.291905E-08	0.287524E-08
0.	0.	0.	0.	0.	0.	0.370043E-06	0.
VULCAN ID = 16							
0.302238E-07	0.295390E-07	0.108767E-07	0.873678E-08	0.632088E-08	0.444418E-08	0.332648E-08	0.327656E-08
0.	0.	0.	0.	0.	0.	0.421693E-06	0.
VULCAN ID = 15							
0.509474E-08	0.502907E-08	0.192306E-08	0.170460E-08	0.140941E-08	0.110963E-08	0.887707E-09	0.867318E-09
0.	0.	0.	0.	0.	0.	0.798113E-07	0.

VULCAN ID = 14
0.122601E-08 0.121893E-08 0.479799E-09 0.465411E-09 0.455785E-09 0.443908E-09 0.423447E-09 0.403793E-09
0. 0. 0. 0. 0. 0. 0.230056E-07

VULCAN ID = 13
0.262193E-05 0.256260E-05 0.943688E-06 0.758215E-06 0.548729E-06 0.385904E-06 0.288892E-06 0.284552E-06
0. 0. 0. 0. 0. 0. 0.365918E-04

VULCAN ID = 21
0. 0. 0.481702E-04 0.483507E-04 0.485680E-04 0.487387E-04 0.488372E-04 0.488327E-04
0. 0. 0. 0. 0. 0. 0.148020E-02

VULCAN ID = 19
0.899276E-03 0.901563E-03 0. 0. 0. 0. 0. 0.
0. 0. 0. 0. 0. 0. 0.686410E-02

VULCAN ID = 20
0.899276E-03 0.901563E-03 0. 0. 0. 0. 0. 0.
0. 0. 0. 0. 0. 0. 0.686410E-02

VULCAN ID = 9
0. 0. 0.735355E-06 0.585230E-06 0.424005E-06 0.298431E-06 0.220330E-06 0.209840E-06
0. 0. 0. 0. 0. 0. 0.130596E-04

VULCAN ID = 10
0. 0. 0.542739E-08 0.355822E-08 0.188173E-08 0.937784E-09 0.532431E-09 0.525376E-09
0. 0. 0. 0. 0. 0. 0.688059E-07

VULCAN ID = 11
0. 0. 0.437833E-10 0.237101E-10 0.918375E-11 0.324948E-11 0.142090E-11 0.145240E-11
0. 0. 0. 0. 0. 0. 0.443487E-09

VULCAN ID = 18
0.877980E-03 0.880588E-03 0. 0. 0. 0. 0. 0.
0. 0. 0. 0. 0. 0. 0.670346E-02

VULCAN ID = 22
0.138714E-03 0.138787E-03 0. 0. 0. 0. 0. 0.
0. 0. 0. 0. 0. 0. 0.105737E-02

CONVERSION RATIO BY MESH INTERVAL AT START OF TIME INTERVAL 1

-0. -0. 0.759312E 00 0.739331E 00 0.733734E 00 0.729275E 00 0.707063E 00 0.664022E 00
0. 0. 0. 0. 0. 0. 0. 0.

CONVERSION RATIO BY MESH INTERVAL AT END OF TIME INTERVAL 1

-0. -0. 0.749893E 00 0.732101E 00 0.728515E 00 0.725613E 00 0.704489E 00 0.661847E 00
0. 0. 0. 0. 0. 0. 0. 0.

BY MESH INTERVAL THE POWER IS

0.462717E 03 0.452355E 03 0.168815E 03 0.135525E 03 0.979981E 02 0.688741E 02 0.515376E 02 0.507576E 02
0. 0. 0. 0. 0. 0. 0.649533E 04

BY MESH INTERVAL THE POWER (NORMALIZED TO VOLUME-AVERAGED POWER) IS

0.271419E 01 0.265340E 01 0.990226E 00 0.794954E 00 0.574833E 00 0.403998E 00 0.302307E 00 0.297731E 00
0. 0. 0. 0. 0. 0. 0.170481E 03

VOLUME AVERAGED EQUILIBRIUM CONCENTRATIONS OF MAJOR FISSION PRODUCTS
IN NUCLEI/BARN*CM IF THE POWER LEVEL OF .649533E 04 WATTS IS MAINTAINED

XENON-135 0.607281E-09
SAMARIUM-149 0.763967E-08
IODINE-135 0.111305E-07
PROMETHIUM-149 0.194277E-07

FISSIONABLE ISOTOPE	INITIAL MASS IN KILOGRAMS	FINAL MASS IN KILOGRAMS	FRACTIONAL POWER
U-235	0.142829E-02	0.140295E-02	0.944316E 00
U-238	0.815713E-01	0.815653E-01	0.483206E-01
U-236	0.	0.149527E-05	0.
PU239	0.	0.518351E-05	0.736294E-02
PU240	0.	0.274243E-07	0.
PU241	0.	0.177500E-09	0.370968E-06

THE FOLLOWING EDIT IS AVERAGED OVER ALL MESH INTERVALS IN WHICH EACH ISOTOPE OCCURS

ISOTOPE DESCR.	N(T)	N(T+1)	N(T+1)/N(T)	POISON FACTOR
HYD -23	0.364272E-01	0.364272E-01	0.100000E 01	0.563034E-01
OKY -24	0.243197E-01	0.243197E-01	0.100000E 01	0.837446E-02
U-235- 5	0.141850E-03	0.138763E-03	0.978237E 00	0.196461E 00
U-238- 7	0.677100E-02	0.677051E-02	0.999927E 00	0.126868E 00
U-236- 6	0.	0.500686E-06	0.	0.599811E-05
PM149-17	0.	0.971242E-08	0.	0.
I-135-16	0.	0.110681E-07	0.	0.
SM149-15	0.	0.209479E-08	0.	0.129656E-02
XE135-14	0.	0.603820E-09	0.	0.186447E-01
FP2 -13	0.	0.960413E-06	0.	0.807945E-03
2ND25-21	0.491500E-04	0.485632E-04	0.988050E 00	0.149980E 00
B10 1-19	0.100000E-02	0.900801E-03	0.900801E 00	0.605550E 01
B10 2-20	0.100000E-02	0.900801E-03	0.900801E 00	0.605550E 01
PU239- 9	0.	0.428466E-06	0.	0.343608E-02
PU240-10	0.	0.225741E-08	0.	0.764434E-05
PU241-11	0.	0.145501E-10	0.	0.153195E-06
SPMAT-18	0.100000E-02	0.879719E-03	0.879719E 00	0.725433E 01
XFIS2-22	0.141850E-03	0.138763E-03	0.978237E 00	0.196461E 00

	INITIAL	FINAL
TOTAL FUEL INVENTORY IN KILOGRAMS	0.829996E-01	0.829789E-01
TOTAL FUEL INVENTORY IN POUNDS	0.182981E 00	0.182935E 00
CONVERSION RATIO	0.202256E 00	0.205006E 00
TOTAL POWER IN WATTS	0.649533E 04	0.642872E 04
MEGAWATT-DAYS/METRIC TON		0.654201E 06

X*E-1 Y*E 1

0. -0.
0.254-0.

0.762 7.499

1.270 7.321

1.850 7.285

2.430 7.256

3.010 7.045

3.590 6.618

3.810 0.

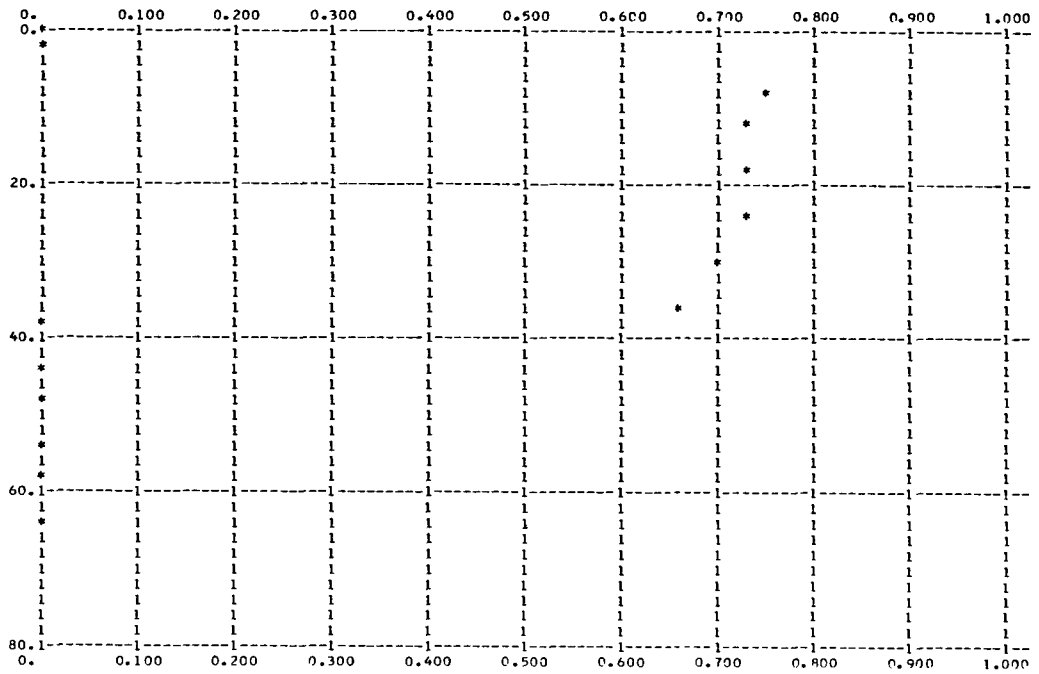
4.310 0.

4.810 0.

5.310 0.

5.810 0.

6.310 0.



CONVERSION RATIO VS. MESH INTERVAL RADIUS

X*E-1 Y*E-0

0. 2.714
0.254 2.653

0.762 0.990

1.270 0.795

1.850 0.575

2.430 0.404

3.010 0.302

3.590 0.298

3.810 0.

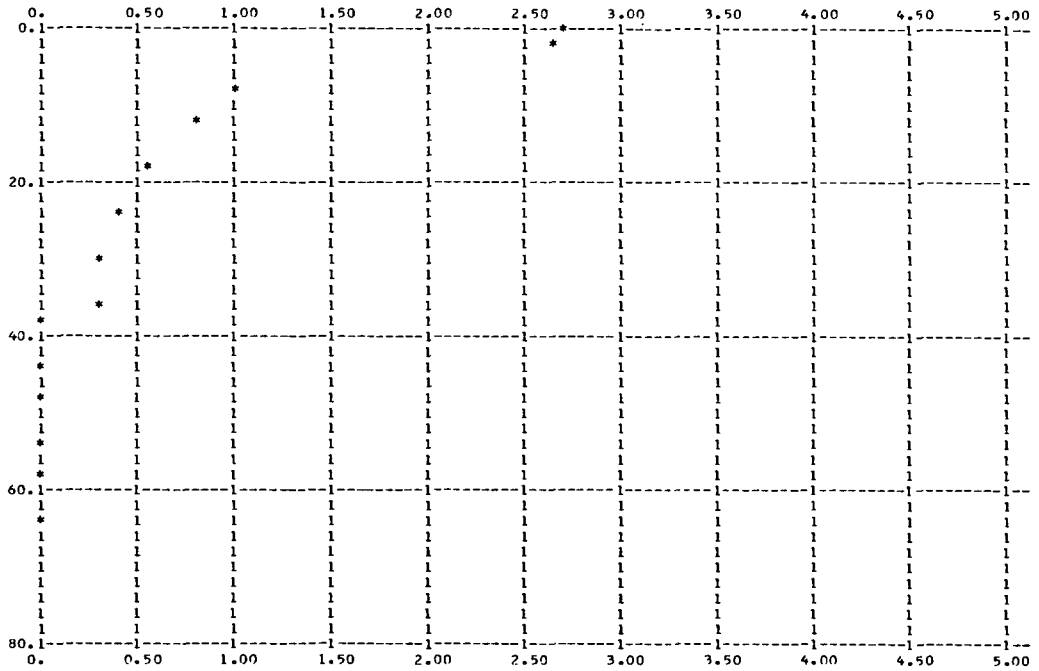
4.310 0.

4.810 0.

5.310 0.

5.810 0.

6.310 0.



NORMALIZED POWER IN INTERVAL VS. MESH INTERVAL RADIUS

***** THIS COMPLETES TIME INTERVAL 1 OF 2

VULCAN DESCRIPTION

VULCAN is a one-dimensional point (mesh interval) depletion or burnup code primarily designed to use the GAM-GATHER-TDSN calculational system and its associated input and output. The input for VULCAN requires geometry and material specifications, macroscopic (atom density in atoms/(b)(cm)) specifications, microscopic cross sections in barns, and the space-energy flux, which is normalized, within VULCAN, to units of neutrons per barn per second. Variable input format options are available for the cross sections and the flux making VULCAN readily adaptable to any space-energy calculation sequence. Thus any reactor configuration, for which geometry and materials can be specified, microscopic cross sections obtained, and fluxes provided, may be treated. Input shortcuts are provided if several configurations, differing only in isotopic densities and flux shape, are considered. In particular, power factors may be used in order to adjust the flux levels. If, in a transverse dimension, a similar material configuration exists, then several transverse dimension regions may be treated sequentially within one VULCAN problem. Thus some measure of two-dimensional depletion is available. The program can treat up to 40 nuclides and 20 energy groups and can accommodate 190 mesh intervals. The mesh intervals may be grouped into 190 zones comprised of 190 distinct materials. These restrictions result because VULCAN is limited to storage in the fast memory of an IBM 7094 computer. However, minor re-programming can greatly increase the range of problems that may be treated. The restrictions on the number of mesh intervals, zones, and materials NRM and energy groups NG may be varied subject to the following condition:

$$52 * NRM + NRM * NG + 112 * NG \leq 16\ 040$$

The COMMON blocks /SET 1/, /SET 2/, and /SET 3/ must be suitably altered in each subroutine and the source decks recompiled. The storage limitation (16 040) is dependent on the storage allocation for internal systems routines.

Description by Subroutine

Main program: VULCAN. - This is the main program and it performs virtually all input-output operations. It controls the flow of the program and performs special edits of power and conversion ratios in the form of plots at the end of each time interval T_1 . The conversion ratio CR_m in any mesh interval m is the ratio of the creation rate C_m to destruction rate D_m of fissionable material in that mesh interval. The equation for CR_m is

$$CR_m = \frac{C_m}{D_m} = \frac{N^1(\sigma_a^1 - \sigma_f^1)\Phi + N^7(\sigma_a^7 - \sigma_f^7)\Phi + N^4(\sigma_a^4 - \sigma_f^4)\Phi + N^{10}(\sigma_a^{10} - \sigma_f^{10})\Phi}{N^3\sigma_a^3\Phi + N^5\sigma_a^5\Phi + N^9\sigma_a^9\Phi + N^{11}\sigma_a^{11}\Phi + N^2\sigma_a^2\Phi \left(\frac{\lambda^2}{\lambda^2 + \sigma_a^2\Phi} \right) + N^8\sigma_a^8\Phi \left(\frac{\lambda^8}{\lambda^8 + \sigma_a^8\Phi} \right)}$$

(16)

The total conversion ratio CR over all mesh intervals NRM is

$$CR = \frac{\sum_{m=1}^{NRM} C_m \cdot (\text{Volume})_m}{\sum_{m=1}^{NRM} D_m \cdot (\text{Volume})_m}$$

New geometry and material specifications are punched by this routine (discussed in the section Subroutine OUTPUT).

Subroutine TABLE. - This subroutine contains tabular information such as, fission product yields, decay constants, atomic weights, thermal ν values (thermal ν = average number of neutrons per thermal fission), and the VULCAN internal identification assignments. It contains a constant, 3.1×10^{10} fissions per thermal watt-second, to be used for absolute flux normalization. The tabular information is taken, for the most part from references 10 to 12. Fission product yields of Xe¹³⁵, I¹³⁵, Sm¹⁴⁹, and Pm¹⁴⁹ from fast and thermal fission of U²³³, U²³⁵, Pu²³⁹, and Pu²⁴¹ are included to the extent they are available. These tabulated yields will be used unless different values are specified in the input. The FORTRAN deck of this subroutine is short; thus any of the tabulated values may be changed and the deck recompiled with little effort.

Subroutine ABSPHI. - Within ABSPHI, prior to the solution of any depletion equations, the arbitrary level input flux is normalized to a specified power. At the end of the depletion time interval the flux is renormalized to the same power before calculating the final conversion ratios and before traversing another depletion time interval.

Subroutine XENON. - The equilibrium concentrations of the four specific fission products (Xe¹³⁵, Sm¹⁴⁹, I¹³⁵, and Pm¹⁴⁹) are calculated here as well as the time after shutdown (at the end of the current time interval T_1) of peak Xe¹³⁵ concentration and its value at that time.

Subroutine SSF. - If isotopic self-shielding factors by energy group are desired, they are read in by this subroutine. The absorption and fission cross sections are multiplied by their respective self-shielding factors.

Subroutines BURNUP and FISEQ. - These two subroutines perform the actual depletion calculation. For each isotope in each mesh interval, the new atom density, fractional change in atom density, fractional power supplied, and a poison factor are calculated. The poison factor for an isotope is the ratio of its macroscopic absorption cross section to the total macroscopic absorption cross section of the fuel. For each mesh interval, the power is calculated; after normalization to the volume-averaged power, a power distribution plot is produced in the main program VULCAN. A fuel inventory is carried out, volume-averaged atom densities obtained, and the conversion ratio determined before and after the depletion time step.

The subroutine allows the concentrations of the four specific fission products, within any mesh interval, to be determined at an arbitrary number of fractional times within any time interval T_1 . For these specific fission products, equilibrium concentrations are determined for each mesh interval and volume averaged over all mesh intervals. For nondepletable regions, the atom densities for the first mesh interval within such a region are carried through as representative of the full region.

Detailed depletion information may be obtained for each mesh interval separately in addition to the information averaged over all mesh intervals. Upon completing the depletion calculation for all mesh intervals, the flux is renormalized (using new atom densities) to the specified power and a new time step T_1 within VULCAN is initiated.

Subroutine OUTPUT. - In a point (mesh interval) depletion program each point within a zone (constant material) will usually be depleted individually. Consequently, each point becomes a distinct material and zone in the next spatial calculation. It is desirable, then, to have some means of minimizing the number of materials or zones. Once all the desired time steps T_1 are completed within VULCAN, the macroscopic absorption cross sections are subjected to two comparative tests. One test determines whether the macroscopic absorption cross section (macs) of each isotope in a mesh interval is, within a predetermined limit, the same as the corresponding isotopic macs in a preceding mesh interval. The other test determines whether the macs of each isotope in a mesh interval constitutes a certain fraction of the total fuel macs in that mesh interval. Each test is performed for each energy group. The mathematical representation of these tests is

Test 1:

$$\left(1 - \frac{\left| \sum_{jm}^k a_{jm} \right|}{\left| \sum_{j(m-n)}^k a_{j(m-n)} \right|} \right) \leq n\epsilon \quad (17)$$

Test 2:

$$\Sigma_{a_{jm}}^k < 10 \epsilon \sum_{i=1}^{NF} \Sigma_{a_{jm}}^i \quad (18)$$

where

$\Sigma_{a_{jm}}^k$ macroscopic absorption cross section for isotope k , energy group j , and mesh interval m

ϵ predetermined limit (see DIFFER in the section INPUT INSTRUCTIONS)

n designates which preceding mesh interval is being compared with mesh interval m

i designates fissionable isotope

If all energy groups for all isotopes satisfy both tests for two or more mesh intervals, the atom densities for these intervals are volume averaged and thus constitute a single new material and zone. After performing these two tests through all mesh intervals, new atom densities for each distinct material are punched. These atom densities are used to obtain new material macroscopic cross sections which will then be used in the spatial calculation (see the section Depletion cycle).

This subroutine, on option, will print out macroscopic absorption cross sections by group, mesh interval, and isotope. Upon return to the main program the geometry and material specifications for input to TDSN are punched corresponding to the new region and material structure. The corresponding output for nondepletable regions maintains the atom densities and mesh structure unaltered and alters, not the material, but its external identification number.

Nuclide List

VID numbers 1 to 17 may be used only for the indicated isotope. If any of these isotopes is not to be considered, its VID number may not be used by another isotope. Nuclides other than the 17 specifically mentioned are assigned VID numbers in unbroken numeric order, beginning with VID = 18. Table II lists the VID number assignments which VULCAN uses.

TABLE II. - NUCLIDE LIST AND VID ASSIGNMENT

Nuclide	VID number
Th ²³²	1
Pa ²³³	2
U ²³³	3
U ²³⁴	4
U ²³⁵	5
U ²³⁶	6
U ²³⁸	7
Np ²³⁹	8
Pu ²³⁹	9
Pu ²⁴⁰	10
Pu ²⁴¹	11
Pu ²⁴²	12
Fission product aggregate	13
Xe ¹³⁵	14
Sm ¹⁴⁹	15
I ¹³⁵	16
Pm ¹⁴⁹	17
Special Materials including additional fission product aggregates	^a 17 + 1 → 17 + J
Burnable Poisons	^a 17 + J + 1 → 17 + J + K
Repeated Fissionable Nuclides	^a 17 + J + K + 1 → 17 + J + K + L
Nondepletable Nuclides	^a 17 + J + K + L + 1 → 17 + J + K + L + M

^aFor a configuration containing J Special Materials, K Burnable Poisons, L Repeated Fissionable Nuclides, and M Nondepletable Nuclides where $J + K + L + M \leq (40 - 17)$.

INPUT INSTRUCTIONS

This section contains the input instructions and explanations of the input parameters. The symbol * after a card number means to use as much of the card or as many cards as necessary.

Card	Format	Variable	Description
1*	I1, 1X, 14A5	TITLE	Title, label, or problem description containing any alphameric information. The number 1 in card column 1 will signify the last title card.
2	7I10	NG	Number of energy groups ≤ 20
		NFG	Number of fast energy groups ≤ 20
		NISOT	Number of isotopes whose depletion and/or buildup is to be considered plus those classified as non-depletable ≤ 40 .
		NMAT	Number of separate materials ≤ 190 (A material is any distinct macroscopic cross section set.)
		NTINC	Number of equal time intervals (DELHR) for which flux normalization and subsequent depletion will be performed.
		NFRACT	Number of fractional steps to be taken within a major time interval (DELHR) for intermediate poison calculations ≤ 10 . NFRACT = 0 if MOREOUT = 0 or 1.
3	7I10	KCELL	= 0 or 1 Normal single depletion $2 \leq KCELL \leq 20$ A series of KCELL successive problems with distinct atom densities and fluxes will be depleted. > 21 A series of KCELL - 20 successive problems differing only in flux level will be depleted (treated by power factors to be supplied).
		NZONR	Number of zones ≤ 190 - within each zone the material is constant.
		NRM	Number of mesh intervals ≤ 190

Card	Format	Variable	Description
		MOROUT	<p>= 0 Only averaged information (always obtained) at the end of depletion sweep will be printed.</p> <p>= 1 Macroscopic absorption cross sections (with self-shielding factors included) will be printed.</p> <p>= 2 Individual mesh interval information will be printed.</p> <p>= 3 Both macroscopic absorption cross sections (with self-shielding factors included) and individual mesh interval information will be printed.</p>
		NUF	<p>= 0 Both $\nu\sigma_f$ and ν will be read in (ν is average number of neutrons per fission - by group and by isotope).</p> <p>= 1 Only σ_f will be read in.</p>
4	7I10	NSPMAT	Number of special materials (see eq. (14)). If more than one fission product aggregate is to be considered, the number must be included in NSPMAT.
		NBPOI	Number of burnable poisons considered
		NFIS	Number of fissionable isotopes for which fission product yields are to be used (indicated by Y in the output) ≤ 10 (NFIS does not include any Repeated Fissionable Nuclides.)
		NYDNUC	Number of fission product nuclides for which yields are to be read in ≤ 20 (Total number whose yields are to be considered is equal to NYDNUC plus the number whose yields are to be taken from subroutine TABLE.)
		NONDPL	Number of nondepletable isotopes (carried through to maintain continuity of the calculational sequence)
		NFIRPT	Number of times that one or more of the twelve fissionable isotopes is repeated. For example, if three distinct cross section sets for U^{235} and two distinct sets for U^{238} are to be used in a single problem, then $NFIRPT = (3 - 1) + (2 - 1) = 3$.

Card	Format	Variable	Description
5	7I10	KGEO	Geometry indicator (one-dimensional): = 1 Slab = 2 Cylinder = 3 Sphere
		KFLUX	Format for flux input: = 1 Binary-TDSN output = 2 Decimal-NRM values for group 1, NRM values for group 2, and so forth (NG sets of NRM values)
		KSSF	Self-shielding factors: = 0 If none = 1 If factors are to be used
		KHAIN	Fission isotope chain applicable: = 0 Both chains = 1 Th ²³² → U ²³⁶ chain = 2 U ²³⁸ → Pu ²⁴² chain
		KFAST	= 0 For thermal reactor = 1 For fast reactor (This option chooses fission product yields for two general reactor classes.)
		KFORM	= 0 KFLUX option is used. = 1 Permits optional FORMAT statement to be read in for cross section input = 2 Permits optional FORMAT statement to be read in for flux input = 3 Permits optional FORMAT statement to be read in for both cross section input and flux input
6	7E10.5	POWER	Total power in watts (thermal power of 1 W = 3.1×10^{10} fissions/sec)
		DISTI	First transverse dimension equals height for cylinder or slab in centimeters (used to calculate power density)
		DIST2	Second transverse dimension equals thickness for slab in centimeters (used to calculate power density)

Card	Format	Variable	Description
		DELHR	Time increment in hours T_1 (Total depletion time is $NTINC \cdot DELHR$.)
		XREMP	Xenon removal rate ξ_x (other than by decay and depletion, e.g., diffusion).
		REMIOD	Iodine removal rate ξ_I
		DIFFER	Averaging criterion for output materials. If $DIFFER = 0$, $DIFFER$ will be set equal to 0.001 (0.1 percent). See ϵ in inequalities (17) and (18).

Card 7 is included only if $NFRACT \neq 0$.

7*	7F10.8	FRACT(I)	Fractional values of DELHR to be used - no restriction on values except that $NFRACT (\leq 10)$ values are necessary.
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Isotope Identification

One card 8 must be included for each of NISOT isotopes. Any isotope may have any sequential ID number (I), but cards 8 must be in numerical order by (I).

8*	4I5, A5	I	Sequential ID number
		VID(I)	VID number (see the section Nuclide List)
		NRPT(I)	= 0 If not a repeated fissionable nuclide = $10 + VID$ If a repeated fissionable nuclide If U^{235} were used twice in addition to the initial occurrence, <u>each</u> repeated time NRPT would be $10 + 5 = 15$. Nuclides for which $NRPT \neq 0$ must be in ascending NRPT number.
		KFIS(I)	= 0 If no fission product yield is to be used or if $NRPT(I) \neq 0$ (Note that, upon completion of the input, the $KFIS(I)$ array will contain NFIS nonzero values.) = 1 If a fission product yield is to be used
		DESCR	Arbitrary alphameric description of VID(I)

Card	Format	Variable	Description
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Geometry and Material Specifications (NZONR Cards 9 Required)

NZONR cards 9 are required. The first card is for zone $K = 1$; the second card is for zone $K = 2$, and so forth; the last card is for zone $K = \text{NZONR}$.

9*	I5, F10.6, I5	NMIR(K)	Number of mesh intervals in zone K
		RAR(K)	External dimension of zone K (cm)
		MIDRZ(K)	Material number corresponding to zone K

If NYDNUC = 0, skip card 10. If NYDNUC \neq 0, NYDNUC cards 10 are required.

10*	2I5, 6F10.8	KRON	= 0 If this isotope is not produced by decay of preceding isotope in NYDNUC series (always 0 for first card in series) = 1 If isotope is capture product of preceding nuclide
		IDFP	VID number of the particular isotope (e. g. , if only Xe^{135} yields are to be read in IDFP = 14)
		YELD(J)	Fractional fission yield of isotope with VID = IDFP. YELD(1) from first nonzero KFIS isotope, YELD(2) from the second nonzero KFIS isotope, and so forth - supplied in the same sequence as (I) on cards 8*. If NFIS \geq 7, continue the yields on the next card (format - 7F10.8).

If NUF = 1, skip cards 11. If NUF = 0, NFIS cards 11 are required.

For each fissionable isotope considered and if NUF = 0, the values of ν are read in for all fast groups. These groups are ordered from high to low. Note that σ_f rather than $\nu\sigma_f$ is used in VULCAN. The card sequence is ordered according to VID number.

11*	7F10.8	XNU(IG)	NFG values of ν in neutrons per fission. Do not include for the NFIRPT isotopes.
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If KFORM = 0 or 2, skip card 12.

12	12A6	FMT(I)	Optional FORMAT statement for cross section input. For the specific format of cards 14, card 12 would read (36X, 2F12.8). Note omission of word FORMAT but the inclusion of parentheses.
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Card	Format	Variable	Description
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Microscopic Cross Sections

Include NISOT sets of cards 13 and 14 in the same sequential order as (I) on cards 8.

13	12A6	DUMMY	Not retained in VULCAN (may be title card for cross sections)
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If KFORM = 1 or 3, read in absorption and fission cross sections for NG groups in specified format (card 12) on cards 14. For each card 13, there must follow NG cards 14.

14*	36X, 2F12.8 (or format indicated on card 12)	ABSIG(IG)	Absorption cross section for group IG in barns
		FUNSIG(IG)	σ_f or $\nu\sigma_f$ cross section in barns (depending on availability) with appropriate values of XNU(IG) on cards 11 and NUF on card 3.

Material Specifications (Atom Densities)

There must be NMAT sets of cards 15 and 16. The NMAT sets must be in numerical order by material number.

15	7I10	NCON	Number of constituent nuclides in this material ≤ 40 (Only those nuclides present at the start of the depletion interval need be included.)
		IDCON(I)	Sequential ID numbers in ascending order (as established by (I) on cards 8) - NCON values. If NCON > 6, continue with 7I10 format on as many cards as are needed.
16*	7F10.8	DENS(I)	Corresponding atom densities for this material (NCON values) in atoms per barn-centimeter

If KFORM = 0 or 1, skip card 17.

17	12A6	FMT(I)	Optional FORMAT statement for flux input
----	------	--------	--

Card	Format	Variable	Description
------	--------	----------	-------------

Fluxes

The fluxes are supplied on cards 18.

18*	7E10.5	FLUX(I)	Average fluxes by mesh interval for each group; NG sets of NRM values (normalized to units of neutrons/(b)(sec) within VULCAN)
-----	--------	---------	--

Power Factors

If $KCELL \leq 20$, skip cards 19 and 20. If $KCELL > 21$, the power factors are supplied on cards 19 and 20.

19	I10	NKOREG	Number of successive problems to be depleted using the same fluxes but adjusted by power factors ≤ 19
20*	7E10.5	CORPOW(I)	Actual, fractional, or relative power for successive problems; NKOREG values; the normalization is to the input, that is, $\frac{\text{POWER}}{\text{CORPOW}(1)} * \text{CORPOW}(I) = \text{actual power in problem } (I)$

Self-Shielding Factors

If $KSSF > 0$, read in the self-shielding factors on cards 21, 22, and 23.

21	I10	NSSF	Total number of nuclides for which self-shielding factors are to be considered ≤ 20
22*	7I10	JSMAT(I)	VID numbers for self-shielded nuclides - NSSF values
23*	7F10.6	SSF(I, J)	Self-shielding factors for NSSF isotopes (I) and NG groups (J); NSSF sets of NG values in the order established by JSMAT(I) (card 22)

If $KCELL > 21$, read in $KCELL - 21$ sets of atom densities and self-shielding factors. Each set of atom densities is composed of the NMAT pairs of cards 15 and 16 and must

be similar to the first set to the extent that only density magnitude changes (i.e., no changes in material constituents) are allowed. Different self-shielding factors may be used for succeeding sets. If $2 \leq KCELL \leq 20$, read in $KCELL - 1$ additional sets of similar atom densities, fluxes, and self-shielding factors (cards 15, 16, 17 (if used before), 18, 21, 22, and 23).

LISTING OF PROGRAM DECKS

The FORTRAN listing of the VULCAN program which follows contains several particular features of the Lewis Monitor System. The subroutine BCREAD (called from main subroutine VULCAN) may be used to read fluxes from binary cards. The subroutine PLOTXY forms plots of two related variables. The following printer carriage control characters have been used as the first character in a format statement:

- J Single space before printing rest of line
- K Double space before printing rest of line
- L Triple space before printing rest of line
- \$ Punch the line instead of print
- * Print and punch this line

For the first three control characters, the character must be counted as one of those being printed, but for the last two, the format is given as if the control characters were not a part of the format statement. The last control character * is actually followed by a blank to make up the complete control symbol. If these characters are not available in the monitor system used, then other output control statements must be used.

```

$IBFTC VULCAN DECK
C
C VULCAN IS A POINT (MESH INTERVAL) DEPLETION CODE
C IT HAS OPTIONAL INPUT FORMATS THAT MAKE IT ADAPTABLE TO
C AN ARBITRARY ONE-DIMENSIONAL SPATIAL CALCULATION
C
C VULCAN USES NO PERIPHERAL STORAGE
C
C WRITTEN IN FORTRAN IV LANGUAGE
C
C JL ANDERSON NUCLEAR SYSTEMS DIVISION
C LEWIS RESEARCH CENTER
C AUGUST 9, 1966
C
COMMON/SET1/ ATDEN(190,40),CONRAT(200),TOPOW(200),VOL(200),RR(200)
1,RATBEF(200)
COMMON/SET2/ FLUX(3800),SANSNU(40,20),ABSIG(40,20),SST(20,20),
1 XNU(12,20)
COMMON/SET3/ NMIR(200),RAUM(200),RAR(200),DELR(200),MIDRZ(400),
1NEMO(200)
COMMON/SET4/ATWGT(20),DECAY(40),THERNU(40),TNAME(40),FRACT(10),
1NRPT(40),IDVULC(40)
COMMON/SET5/ VULABS(40),VULFIS(40),VULAMF(40),ADPREV(40),IDYLD(10)
1,Y(20,12),KKUN(20)
COMMON/SET6/ NG,NM,NISOT,TOTVOL,FIWATT,POWER,AVPOW,FNDRM
COMMON/SET7/NFIS,NSPMAT,NBPOI,NFRACT,NYDNUC,NFIRPT
COMMON/SET8/ DELHR,KHAIN,AVOGAD,NZONR,MOROUT,MOOD,DIFFER,NONDPL
COMMON/SET9/ KTIM,NTINC,KRELL,KCELL
COMMON/SET10/ KOMP,KONG,KOG,LAND,DELSEC,MESH
COMMON/SET11/ NSSF,JSMAT(20)
C
C DIMENSION CORPOW(20),RAPOF(20),TITLE(14),YELD(10),IDCON(40),
1NUM(14),KFIS(40),DENS(40),IDFP(20),PI(15),UNAME(10)
DIMENSION FUNSIG(40,20)
DIMENSION FMT(12)
C
C EQUIVALENCE (FUNSIG,SANSNU)
C
C
C READ IN INPUT DATA
C
14 WRITE(6,107)
2 READ(5,100) IJK,(TITLE(I),I=1,14)
WRITE(6,159) (TITLE(I),I=1,14)
IF(IJK.NE.1) GO TO 2
WRITE(6,110)
READ(5,101) NG,NFG,NISOT,NMAT,NTINC,NFRACT,KCELL
WRITE(6,101) NG,NFG,NISOT,NMAT,NTINC,NFRACT,KCELL
WRITE(6,111)
READ(5,101) NZONR,NRM,MOROUT,NUF
WRITE(6,101) NZONR,NRM,MOROUT,NUF
WRITE(6,112)
READ(5,101) NSPMAT,NBPOI,NFIS,NYDNUC,NONDPL,NFIRPT
WRITE(6,101) NSPMAT,NBPOI,NFIS,NYDNUC,NONDPL,NFIRPT
WRITE(6,113)
READ(5,101) KGEO,KFLUX,KSSF,KHAIN,KFAST,KFORM
WRITE(6,101) KGEO,KFLUX,KSSF,KHAIN,KFAST,KFORM
WRITE(6,114)
READ(5,102) POWER,DIST1,DIST2,DELHR,XREMP,REMIOD,DIFFER
WRITE(6,141) POWER,DIST1,DIST2,DELHR,XREMP,REMIOD,DIFFER
IF(NFRACT.EQ.0) GO TO 3
READ(5,104) (FRACT(JK),JK=1,NFRACT)
3 WRITE(6,115)
CALL TABLE (NISOT,FIWATT,AVOGAD,KFAST)
DECAY(14)=DECAY(14)+XREMP
DECAY(40)=DECAY(16)+REMIOD
C DECAY(40) IS ACTUALLY DECAY(16) FOR THE I-135 EQUATION
C DECAY(16)=DECAY(16)-REMIOD
C DECAY(16) IS FOR THE XE-135 EQUATION
C DO 1 KISS=1,NISOT
C I REFERS TO MACGG SEQUENCE BUT CARDS (NISOT) MAY BE IN ANY ORDER
READ(5,143) I,IDVULC(I),NRPT(I),KFIS(I),DESCR
ISM=IDVULC(I)
TNAME(ISM)=DESCR
IF(KFIS(I).EQ.0) GO TO 4
WRITE(6,145) I,IDVULC(I),NRPT(I),TNAME(ISM)
GO TO 1
4 WRITE(6,144) I,IDVULC(I),NRPT(I),TNAME(ISM)
1 CONTINUE
READ(5,103) (NMIR(I),RAR(I),MIDRZ(I),I=1,NZONR)
WRITE(6,116)
WRITE(6,119)
IGO=NRM+1
DO 21 I=1,NZONR
IEND=IGO+NMIR(I)-1
DO 22 J=IGO,IEND
22 MIDRZ(J)=MIDRZ(I)
21 WRITE(6,138) NMIR(I),RAR(I),(MIDRZ(K),K=IGO,IEND)
C
C MESH INTERVAL MANIPULATION
C
11 PI=3.1415927
DELR(1)=0.0
DELR(1)=RAR(1)
DO 12 I=2,NZONR
DELR(I)=0.0
12 DELR(I)=RAR(I)-RAR(I-1)
DO 13 I=1,NZONR
VMI=0.0
VMI=NMIR(I)

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13 DELR(I)=DELR(I)/VMI
   RUN=0.0
   RR(1)=0.0
   K=1
   DO 17 J=1,NZONR
     NMES=NHIR(J)
     DO 17 I=1,NMES
       K=K+1
       RR(K)=0.0
17  RR(K)=RR(K-1)+DELR(J)
   RUN=RR(NRM+1)
9  CONTINUE

C
C  VOLUME CALCULATION
C
   NM=NRM
   DO 20 JK=1,NM
20  VOL(JK)=0.0
     NRMPI=NRM+1
     GU TO (30,40,50),KGE0
30  VOLT=DIST1*DIST2*RUN
     RAUM(1)=RAR(1)*DIST2*DIST1
     DO 23 NIL=2,NZONR
       RAUM(NIL)=0.0
23  RAUM(NIL)=(RAR(NIL)-RAR(NIL-1))*DIST2*DIST1
     DO 31 K=2,NRMPI
31  VOL(K-1)=(RR(K)-RR(K-1))*DIST2*DIST1
     GO TO 60
40  VOLT=PI*RUN*RUN*DIST1
     RAUM(1)=PI*RAR(1)*RAR(1)*DIST1
     DO 35 MIL=2,NZONR
       RAUM(MIL)=0.0
35  RAUM(MIL)=PI*(RAR(MIL)-RAR(MIL-1))*RAR(MIL-1)*DIST1
     DO 41 K=2,NRMPI
41  VOL(K-1)=PI*(RR(K)-RR(K-1))*RAR(K-1)*DIST1
     GO TO 60
50  VOLT=(4.*PI/3.)*(KUN**3)
     RAUM(1)=(4.*PI/3.)*(RAR(1)**3)
     DO 57 JIL=2,NZONR
       RAUM(JIL)=0.0
57  RAUM(JIL)=(4.*PI/3.)*(RAR(JIL)**3-RAR(JIL-1)**3)
     DO 51 K=2,NRMPI
51  VOL(K-1)=(4./3.)*PI*(RR(K)**3-RR(K-1)**3)
60  TOTVOL=0.0
     DO 52 LBJ=1,NM
52  TOTVOL=TOTVOL+VOL(LBJ)
     WRITE(6,117)
     WRITE(6,139) (VOL(K),K=1,NM),TOTVOL
     WRITE(6,150) (RAUM(KUK),KUK=1,NZONR)
     QUOT=VOLT/TOTVOL
     IF(ABS(1.-QUOT).LT..001) GO TO 215
     WRITE(6,133) TOTVOL,VOLT

C
C  READ IN MORE INPUT DATA
C
C  FISSION INFORMATION
C
215 WRITE(6,107)
     DO 74 I=1,NYDNUC
       IF(NYDNUC.EQ.0) GO TO 230
       IF(NFIS.GT.6) GO TO 5
       READ(5,131) KKON(I),IDFP(I),(YELD(J),J=1,NFIS)
       GO TO 6
5      READ(5,131) KKON(I),IDFP(I),(YELD(J),J=1,6)
       READ(5,104) (YELD(K),K=7,NFIS)
6      IJK=IDFP(I)-12
C      IDFP IS IN VULCAN SEQUENCE
230 DO 70 J=1,NFIS
     IFIS=0
     DO 73 ISEU=1,NISOT
       IFIS=IFIS+KFIS(ISEU)
       IF(IFIS.NE.J) GO TO 73
       IDYLD(J)=IDVULC(ISEU)
       IDT=IDYLD(J)
       IF(NYDNUC.NE.0) Y(IJK,IDT)=YELD(J)
       QNAME(J)=TNAME(IDT)
       GO TO 70
73  CONTINUE
70  CONTINUE
/4  CONTINUE
75  WRITE(6,124) (IDYLD(JA),JA=1,NFIS)
     WRITE(6,147) (QNAME(JA),JA=1,NFIS)
     NUG=NSPMAT+17
     DO 55 NASA=1,NISOT
       NAV=IDVULC(NASA)
       IF(NAV.LE.12.OR.NAV.GT.NUG) GO TO 55
       NUT=NAV-12
     DO 56 JET=1,NFIS
       ID=IDYLD(JET)
56  YELD(JET)=Y(NUT,ID)
     WRITE(6,119)
     WRITE(6,146) NAV,TNAME(NAV),(YELD(JJ),JJ=1,NFIS)
55  CONTINUE
     IF(NUF.EQ.1) GO TO 42
     WRITE(6,109)

C
C  NU IN VULCAN ID ORDER
C
     DO 79 J=1,12
     DO 221 MIG=1,NFG

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221 XNU(J,MIG)=0.0
    NUM(J)=J
    DO 220 NAX=1,NISOT
      IF(J.EQ.IDVULC(NAX)) READ(5,104) (XNU(J,IG),IG=1,NFG)
220 CONTINUE
79 CONTINUE
    WRITE(6,125) (NUM(J),J=1,12)
    WRITE(6,119)
    DO 76 IG=1,NFG
76 WRITE(6,140) IG,(XNU(J,IG),J=1,12)
    IF(KFAST.EQ.0) WRITE(6,142) (THERNU(J),J=1,12)
42 WRITE(6,107)

C
C CROSS SECTIONS
C
    IF(KFORM.EQ.1.OR.KFORM.EQ.3) READ(5,135) (FMT(I),I=1,12)
    DO 67 ISOT=1,NISOT
      MACGG SEQUENCE
      ID=IDVULC(ISOT)
      KID=ID
      READ(5,135) DUMMY
      IF(KFORM.EQ.0.OR.KFORM.EQ.2) GO TO 200
      DO 201 IG=1,NG
201 READ(5,FMT) ABSIG(ID,IG),FUNSIG(ID,IG)
      GO TO 202
200 DO 65 IG=1,NG
      READ(5,105) ABSIG(ID,IG),FUNSIG(ID,IG)
      READ(5,135) DUMMY
65 CONTINUE
202 WRITE(6,108)
    WRITE(6,121) ISOT,ID
    WRITE(6,122)
    WRITE(6,139) (ABSIG(ID,IG),IG=1,NG)
    IF(ID.GT.12.AND.NRPT(ISOT).EQ.0) GO TO 67
    IF(NUF.EQ.1) GO TO 300
    WRITE(6,123)
    WRITE(6,139) (FUNSIG(ID,IG),IG=1,NG)
300 GNU=1.
    DO 68 IG=1,NG
    IF(NUF.EQ.1) GO TO 68
    IF(FUNSIG(ID,IG).EQ.0.) GO TO 68
    IF(NRPT(ISOT).EQ.0) GO TO 69
    KID=NRPT(ISOT)-10
69 CONTINUE
    IF(IG.GT.NFG) GO TO 72
    GNU=XNU(KID,IG)
    IF(GNU.NE.0.) GO TO 68
    GNU=2.5
    WRITE(6,153) ID,IG
    GO TO 68
72 GNU=THERNU(KID)
    IF(GNU.EQ.0.) GNU=1.
68 SANSNU(ID,IG)=FUNSIG(ID,IG)/GNU
    WRITE(6,134)
    WRITE(6,139) (SANSNU(ID,IG),IG=1,NG)
67 CONTINUE

C
C ATOM DENSITIES
C
    LUNE=0
    LUX=0
    KANT=0
    IF(KCELL.LE.20) GO TO 207
    KANT=1
    KCELL=KCELL-20
207 DO 208 NSEL=1,KCELL
    KTIM=1
    LOX=LOX+1
    KRELL=NSEL
    IF(NSEL.GT.1) WRITE(6,157) NSEL,KCELL
61 DO 62 J=1,NMAT
    DO 64 K=1,40
64 DENS(K)=0.0
    READ(5,101) NCON,(IDCON(I),I=1,NCON)
    MACGG SEQUENCE IDCUN
    READ(5,158) (DENS(I),I=1,NCON)
    LADD=0
    DO 53 K=1,NZONR
    MESH=0
    MAT=MIDRZ(K)
    LADD=LADD+NMIR(K)
    IF(MAT.NE.J) GO TO 53
    MESH=LADD-NMIR(K)
    NMES=NMIR(K)
    DO 53 L=1,NMES
    MESH=MESH+1
    DO 235 NEQ=1,40
235 ATDEN(MESH,NEQ)=0.0
    DO 54 KKK=1,NCON
    IAGO=IDCON(KKK)
    ID=IDVULC(IAGO)
54 ATDEN(MESH,ID)=DENS(KKK)
53 CONTINUE
62 CONTINUE
    NGOTO=0
    NST=0
    WRITE(6,107)
    MUSS=0

```

```

36 NST=NGOTO+1
   IF(NMAT.GT.(NGOTO+7)) GO TO 37
   NGOTO=NMAT
   GO TO 45
37 NGOTO=NGOTO+7
   WRITE(6,119)
45 DO 38 KIX=1,7
   NUM(KIX)=0
   NUM(KIX)=NST+KIX-1
   IF(NUM(KIX).GT.NMAT) GO TO 330
   KRAD=0
   DO 310 JKT=1,NZONK
   KRAD=KRAD+NMIR(JKT)
   IF(MIDRZ(JKT).EQ.NUM(KIX)) GO TO 320
310 CONTINUE
320 NUM(KIX+7)=KRAD-NMIR(JKT)+1
   GO TO 38
330 NUM(KIX)=0
38 CONTINUE
   LARAY=NGOTO-NST+1
   IF(NGOTO.GT.7) WRITE(6,119)
   WRITE(6,118) (NUM(K),K=1,7)
   DO 66 MV=1,NISDT
   JVV=IDVULC(MV)
   WRITE(6,119)
   DO 340 NVT=1,LARAY
   LOCUS=NUM(NVT+7)
340 CONKAT(NVT)=ATDEN(LOCUS,JV)
   WRITE(6,120) JV,TNAME(JV),(CONKAT(J),J=1,LARAY)
66 CONTINUE
   IF(NMAT.GT.NGOTO) GO TO 36
   WRITE(6,107)

C
C   FLUX
C
   LST=0
   LSP=0
   IF(NSEL.GT.1.AND.KANT.EQ.1) GO TO 88
   IST=0
   ISP=0
   KST=0
   KSP=0
204 IF(KFORM.LT.2) GO TO (81,82), KFLUX
   READ(5,100) (FMT(I),I=1,12)
   DO 203 IG=1,NG
   IST=ISP+1
   ISP=IST+NM-1
203 READ(5,FMT) (FLUX(ICON),ICON=IST,ISP)
   GO TO 87
81 DO 77 IG=1,NG
   IGO=NM*(IG-1)+1
   ISTOP=IGO+NM-1
   CALL 8CREAD(FLUX(IGO),FLUX(ISTOP))
77 CONTINUE
   GO TO 87
82 DO 86 IG=1,NG
   IST=ISP+1
   ISP=IST+NM-1
86 READ(5,102) (FLUX(ICON),ICON=IST,ISP)
87 CONTINUE
   IF(LUNE.EQ.0) WRITE(6,126)
   IF(KANT.EQ.1) WRITE(6,129)
   IF(KANT.EQ.0.AND.KCELL.GT.0) WRITE(6,130) NSEL
   DO 80 IG=1,NG
   WRITE(6,127) IG
   KST=KSP+1
   KSP=KST+NM-1
80 WRITE(6,139) (FLUX(KCON),KCON=KST,KSP)
88 CALL ABSPHI
   KDOL=0
   IF(KANT.EQ.0) GO TO 206
   IF(NSEL.GT.1) GO TO 212
   LUNE=LUNE+1
   READ(5,106) NKOREG
C   ACTUAL,FRACTIONAL OR RELATIVE POWER MAY BE READ IN
   READ(5,102) (CORPDW(KIP),KIP=1,NKOREG)
   WRITE(6,154) (CORPDW(KIP),KIP=1,NKOREG)
   DO 214 KRAT=1,NKOREG
214 RAPOF(KRAT)=CORPDW(KRAT)/CORPDW(1)
   WRITE(6,128) (RAPOF(JOE),JOE=1,NKOREG)
212 MG=NM*NG
   VIET=1.0
   IF(NSEL.GT.1) VIET=RAPOF(LOX-1)
   DO 205 KK=1,MG
205 FLUX(KK)=FLUX(KK)*RAPOF(LOX)/VIET
   WRITE(6,160) RAPOF(LOX)
   WRITE(6,126)
   WRITE(6,130) NSEL
   DO 43 IGY=1,NG
   WRITE(6,127) IGY
   LST=LSP+1
   LSP=LST+NM-1
43 WRITE(6,139) (FLUX(NAM),NAM=LST,LSP)
206 DO 209 NTM=1,NTINC
   IF(KSSF.GT.0) CALL SSF(NG,NTM)
   KTIM=NTM+1

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89 CALL HURNUP
   IF (NTM.NE.NTINC) GO TO 90
99 CALL OUTPUT
C
C      CALCULATION OF DIMENSION SPECIFICATIONS
C
      NET=1
      WRITE(6,148)
      DO 92 K=1,MOOD
      NET=NEMO(K)+NET
      RAR(K)=RR(NET)
      PUNCH 103, NEMO(K),RAR(K),K
92  WRITE(6,149) NEMO(K),RAR(K),K
90  P(1)=NM
      CALL PLOTXY(RR,CONRAT,64,P)
      WRITE(6,136)
      CALL PLOTXY(RR,TOPW,64,P)
      WRITE(6,137)
      WRITE(6,107)
      WRITE(6,156) NTM,NTINC
209 CONTINUE
208 CONTINUE
91 GO TO 14
100 FORMAT(11,1X,14A5)
101 FORMAT(7F10)
102 FORMAT(7E10.5)
103 FORMAT(15,F10.6,15)
104 FORMAT(7F10.8)
105 FORMAT(36X,2E12.6)
106 FORMAT(110,6F10.7)
107 FORMAT(1H1)
108 FORMAT(1HK)
109 FORMAT(1HL)
110 FORMAT(1HL,7X,2HNG,8X,3HNFG,6X,5HNISOT,5X,4HNMAT,6X,5HNTINC,4X,
16HNFRACT,4X,5HKCELL)
111 FORMAT(1HL,6X,5HNZONR,5X,3HNRM,6X,6HMOROUT,6X,3HNUP)
112 FORMAT(1HL,5X,6HNSPMAT,5X,5HNBPOI,5X,4HNFIS,5X,6HNYDNUC,4X,6HNONDP
1L,4X,6HNFIRPT)
113 FORMAT(1HL,6X,4HKGEO,6X,5HKFLUX,5X,4HKSSF,6X,5HKHAIN,5X,5HKFAST,
15X,5HKFORM)
114 FORMAT(1HL,9X,5HPOWER,6X,5HFIRST,6X,6HSECOND,8X,4HTIME,11X,5HXENON
1,10X,6HINDINE/11X,2HIN,7X,9HDISTANCE,4X,8HDISTANCE,4X,10H INCREME
2NT,7X,7HREMOVAL,9X,7HREMOVAL,5X,6HDIFFER/10X,5HWATTS,6X,4H(CM),
310X,6H(CM),5X,
4 8H (HOURS),4X,13H PROBABILITY,5X,11HPROBABILITY)
115 FORMAT(1HL,4X,10HSEQUENTIAL,7H VULCAN,8H NRPT,7HISOTOPE/
18X,2HID,8X,2HID,11X,5HLABEL)
116 FORMAT(1HL,12H NMIR(I),12H RAR(I),23H MIDRZ(I)-
1MAP)
117 FORMAT(1HL,15H MESH VOLUMES//)
118 FORMAT(8H VULCAN,11H ISOTOPE,7(13H MATERIAL )/4X,2HID,5X,
15HLABEL,8X,7(12,11X))
119 FORMAT(1HJ)
120 FORMAT(15,6X,A5,3X,7E13.5)
121 FORMAT(17H SEQUENTIAL ID = ,12,15H VULCAN ID = ,12)
122 FORMAT(1HK,34H ABSORPTION BY GROUP (HIGH TO LOW))
123 FORMAT(34H NU*FISSION BY GROUP (HIGH TO LOW))
124 FORMAT(10H FISSION,6X,10(10HYIELD )/15H PRODUCT,10(
14HFROM,13,3X))
125 FORMAT(7H GROUP,12(8H NU FOR)/7X,12(5H VID,13))
126 FORMAT(31H FLUXES BY GROUP AND MESH POINT)
127 FORMAT(1HJ,8H GROUP,12)
128 FORMAT(1HK,35H THESE ARE THE RADIAL POWER FACTORS/(8E14.6))
129 FORMAT(24H FOR THE BASIC CELL)
130 FORMAT(22H FOR CELL NUMBER,12)
131 FORMAT(2I5,6F10.8)
133 FORMAT(1HL,13H VOLUME SUM,F13.4,25H NOT EQUAL F(DIST1,DIST2),F13
1.4)
134 FORMAT(33H FISSION BY GROUP (HIGH TO LOW))
135 FORMAT(12A6)
136 FORMAT(2HPL,55X,41HCONVERSION RATIO VS. MESH INTERVAL RADIUS)
137 FORMAT(2HPL,55X,53HNORMALIZED POWER IN INTERVAL VS. MESH INTERVAL
1RADIUS)
138 FORMAT(110,7X,F8.4,9X,25(13)/34X,25(13))
139 FORMAT(8E14.6)
140 FORMAT(15,2X,12F8.3)
141 FORMAT(7X,E10.5,F11.5,F12.5,F14.2,F16.7,F16.7,F11.5)
142 FORMAT(7H T,12F8.3)
143 FORMAT(4I5,A5)
144 FORMAT(2I10,17,4X,A5)
145 FORMAT(2I10,2H Y,15,4X,A5)
146 FORMAT(15,2H (,A5,1H),F9.5,9(F10.5))
147 FORMAT(15X,10(1H(,A5,1H),3X))
148 FORMAT(1HL,41H DIMENSION SPECIFICATION INPUT TO TDSN//)
149 FORMAT(115,F10.6,15)
150 FORMAT(1HL,15H ZONE VOLUMES// (8E14.6))
152 FORMAT(F15.4)
153 FORMAT(1HL,10(1H+),61H A NON-ZERO FISSION CROSS SECTION HAS BEEN
1SUPPLIED FOR VID,12/1X,10(1H+),8H GROUP,12,54H BUT NO NON-ZERO
2NU VALUE IS AVAILABLE. THEREFORE THIS/1X,10(1H+),41H PARTICULAR N
3U HAS BEEN SET EQUAL TO 2.5)
154 FORMAT(1HK,31H THE POWER FOR EACH CORE REGION/(8E14.6))
156 FORMAT(1H2,10(1H*),31H THIS COMPLETES TIME INTERVAL,12,4H OF,12
1)
157 FORMAT(1HL,10(1H*),62H THE FOLLOWING ATOM DENSITIES AND FLUXES AR
1E FOR CELL NUMBER,12,4H OF,12)
158 FORMAT(7E10.6)
159 FORMAT(2X,14A5)
160 FORMAT(1HK,38H THE POWER FACTOR FOR THIS CELL IS ,F10.7)
      END

```



```

C      Y(1,5)=.9260
C      Y(1,9)=.9175
30    ATWGT(1)=232.111
      ATWGT(2)=233.114
      ATWGT(3)=233.112
      ATWGT(4)=234.114
      ATWGT(5)=235.117
      ATWGT(6)=236.120
      ATWGT(7)=238.125
      ATWGT(8)=239.123
      ATWGT(9)=239.127
      ATWGT(10)=240.129
      ATWGT(11)=241.131
      ATWGT(12)=242.134
      DO 90 LBJ=1,NLSOT
      DECAY(LBJ)=0.0
90    THERNU(LBJ)=0.0
      THERNU(1)=2.6
      THERNU(3)=2.503
      THERNU(5)=2.494
      THERNU(7)=2.43
      THERNU(9)=2.60
      THERNU(11)=2.882
      THERNU(13)=2.871
      THERNU(15)=2.98
      THERNU(17)=2.969
      FIWATT=3.10 E10
      AVOGAD=6.0247 E23
      DECAY(2)=2.928 E-7
      DECAY(8)=3.410 E-6
      DECAY(11)=1.650 E-9
      DECAY(14)=2.10 E-5
      DECAY(16)=2.87 E-5
      DECAY(17)=3.63 E-6
      RETURN
      END

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ANL5800
GATHER

ANL5800

NUD V1N5
NUD V1N5

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SIBFTC ABFLUX DECK
      SUBROUTINE ABSPHI
      COMMON/SET1/ ATDEN(190,40),CONRAT(200),TOPOW(200),VOL(200),RR(200)
      1,RATBEF(200)
      COMMON/SET2/ FLUX(3800),SANSNU(40,20),ABSIG(40,20),SST(20,20),
      1 XNU(12,20)
      COMMON/SET3/ NMIR(200),RAUM(200),RAK(200),DELR(200),MIDRZ(400),
      1NEMO(200)
      COMMON/SET4/ATWGT(20),DECAY(40),THERNU(40),TNAME(40),FRACT(10),
      1NKPT(40),IDVULC(40)
      COMMON/SET5/ VULABS(40),VULFIS(40),VULAMF(40),ADPREV(40),IDYLD(10)
      1 ,Y(20,12),KRON(20)
      COMMON/SET6/ NG,NM,NISOT,TOTVOL,FIWATT,POWER,AVPOW,FNORM
      COMMON/SET9/ KTIM,NTINC,KRELL,KCELL
C
C      CALCULATES ABSOLUTE FLUX NORMALIZATION FACTOR
C
      107 FORMAT(1H1)
      132 FORMAT(1HL,45H          FOR THE START OF TIME INTERVAL NUMBER ,I2,
      14H OF ,I2)
      134 FORMAT(1HK,17H          CELL NUMBER ,I2,4H OF ,I2)
      135 FORMAT(1HL,29H          FLUX NORMALIZATION FACTOR ,E12.6, 48H CONVERTS T
      10 ABSOLUTE FLUX IN NEUTRONS/BARN*SEC/20X,22H FOR A POWER LEVEL OF
      2,E12.6,7H WATTS)
      136 FORMAT(1HL,30(3H**))
      IF(KTIM.EQ.1) WRITE(6,107)
      VOLPGR=0.0
      FISPHI=0.0
      DO 10 IG=1,NG
      FISPGR=0.0
      DO 9 IM=1,NM
      FISMAL=0.0
      LOC=NM*(IG-1)+IM
      DO 8 IMAT=1,NISOT
      ID=IDVULC(IMAT)
      IF(SANSNU(ID,IG).EQ.0..OR.ATDEN(IM,ID).EQ.0.) GO TO 8
      FISMAL=FISMAL+ATDEN(IM,ID)*SANSNU(ID,IG)*FLUX(LOC)
      IF(VOL(IM).LT.0.) GO TO 8
      VOL(IM)=-VOL(IM)
      VOLPGR=VOLPGR-VOL(IM)
      8 CONTINUE
      IF(VOL(IM).GT.0.) GO TO 9
      FISPGR=FISPGR-FISMAL*VOL(IM)
      9 CONTINUE
      FISPHI=FISPHI+FISPGR
      10 CONTINUE
      FISPHI=FISPHI/VOLPGR
      DO 6 K=1,NM
      IF(VOL(K).GE.0.) GO TO 6
      VOL(K)=-VOL(K)
      6 CONTINUE
      FNORM=(FIWATT*POWER/(VOLPGR*FISPHI))*10.**(-24)
      WRITE(6,136)
      WRITE(6,132) KTIM,NTINC
      WRITE(6,135) FNORM,POWER
      IF(KCELL.GT.0) WRITE(6,134) KRELL,KCELL
      WRITE(6,136)
      NGM=NG*NM
      DO 11 LOC=1,NGM
      11 FLUX(LOC)=FLUX(LOC)*FNORM
      AVPOW=POWER/VOLPGR
      RETURN
      END

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SIBFTC TXENON DECK
SUBROUTINE XENON (GAINXE,GAINSM,GAINIO,GAINPM,AVGXE,AVGSM,AVGIO,
1AVGPM)
COMMON/SET1/ ATDEN(190,40),CONRAT(200),TOPOW(200),VOL(200),RR(200)
1,RATBEF(200)
COMMON/SET2/ FLUX(3800),SANSNU(40,20),ABSIG(40,20),SST(20,20),
1 XNU(12,20)
COMMON/SET3/ NMIR(200),RAUM(200),RAR(200),DELK(200),MIDKZ(400),
1NEMO(200)
COMMON/SET4/ATWGT(20),DECAY(40),THERNU(40),TNAME(40),FRACT(10),
1NRPT(40),IDVULC(40)
COMMON/SET5/ VULABS(40),VULFIS(40),VULAMF(40),ADPREV(40),IDYLD(10)
1,Y(20,12),KRON(20)
COMMON/SET6/ NG,NM,NISOT,TOTVOL,FIWATT,POWER,AVPOW,FNORM
COMMON/SET7/NFIS,NSPMAT,NBPOI,NFRACT,NYDNUC,NFIRPT
COMMON/SET8/ DELHR,KHAIN,AVOGAD,NZONR,MOROUT,MODU,DIFFER,NUNDPL
COMMON/SET10/ KOMP,KONG,KUG,LAND,DELSEC,MESH
C
C CALCULATES EQUILIBRIUM POISON CONCENTRATION AND TIME AFTER
C SHUTDOWN OF MAXIMUM XENON CONCENTRATION
C
101 FORMAT(73H EQUILIBRIUM CONCENTRATIONS OF MAJOR FISSION PRODUCTS
11N NUCLEI/BARN*CM/33H IF THE CURRENT POWER LEVEL OF ,E10.5,21H
2WATTS IS MAINTAINED)
102 FORMAT(1HK,71H TIME AFTER SHUTDOWN UNTIL MAXIMUM XENON CONCENTR
1ATION IS REACHED = ,F6.2,7H HOURS)
103 FORMAT(1HK,14X,14HXENON-135 ,E20.6/ 15X,14HSAMARIUM-149 ,E20.
16/ 15X,14HIODINE-135 ,E20.6/ 15X,14HPROMETHIUM-149,E20.6)
104 FORMAT(14H EQUILIBRIUM ,4E15.6)
105 FORMAT(14H,51H ** IT SEEMS THE FLUX LEVEL ISN'T HIGH ENOUGH FOR
144H ** A VALID TIME AFTER SHUTDOWN CALCULATION)
106 FORMAT( 55H THE XENON-135 CONCENTRATION (PEAK) AT THIS TIME
11S ,E12.6)
C
C
EQIOD=GAINIO/(DECAY(16)+VULABS(16))
EQXEN=(GAINXE+DECAY(16)*EQIOD)/(DECAY(14)+VULABS(14))
EQPRN=GAINPM/(DECAY(17)+VULABS(17))
EQSAM=(GAINSM+DECAY(17)*EQPRN)/VULABS(15)
IF(MOROUT.LT.2) GO TO 60
IF(NFRACT.EQ.0) GO TO 30
WRITE(6,104) EQXEN,EQSAM,EQIOD,EQPRN
GO TO 40
30 WRITE(6,101) POWER
WRITE(6,103) EQXEN,EQSAM,EQIOD,EQPRN
40 TEMP=1.-(DECAY(14)-DECAY(16))*EQXEN/(DECAY(16)*EQIOD)
IF(TEMP.LT.0.) GO TO 10
TMAX=ALOG(TEMP*DECAY(14)/DECAY(16))/(DECAY(14)-DECAY(16))/3600.
GO TO 20
10 TMAX=-1.
20 WRITE(6,102) TMAX
IF(TMAX.LT.0..AND.TMAX.NE.-1.) WRITE(6,105)
TAFSEC=3600.*TMAX
CONMAX=(DECAY(16)*EQIOD/(DECAY(14)-DECAY(16)))*(EXP(-TAFSEC*DECAY(
116))-EXP(-TAFSEC*DECAY(14)))+EQXEN*EXP(-TAFSEC*DECAY(14))
WRITE(6,106) CONMAX
60 AVGXE=AVGXE+EQXEN*VOL(MESH)
AVGSM=AVGSM+EQSAM*VOL(MESH)
AVGIO=AVGIO+EQIOD*VOL(MESH)
AVGPM=AVGPM+EQPRN*VOL(MESH)
50 RETURN
END

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$IBFTC SELSH DECK
      SUBROUTINE SSF(NG,NTM)
C
C      CALCULATES SSF AND ADJUSTS CROSS SECTIONS
C
      COMMON/SET1/ ATDEN(190,40),CONRAT(200),TOPOW(200),VOL(200),KR(200)
1,RATBEF(200)
      COMMON/SET2/ FLUX(3800),SANSNU(40,20),ABSIG(40,20),SST(20,20),
1 XNU(12,20)
      COMMON/SET3/ NMIR(200),KAUM(200),RAR(200),DELR(200),MIDRZ(400),
1NEMU(200)
      COMMON/SET4/ATWGT(20),DECAY(40),THERNU(40),TNAME(40),FRACT(10),
1NRPT(40),IDVULC(40)
      COMMON/SET5/ VULABS(40),VULFIS(40),VULAMF(40),ADPREV(40),IDYLD(10)
1 ,Y(20,12),KRON(20)
      COMMON/SET11/ NSSF,JSMAT(20)
100 FORMAT(7I10)
101 FORMAT(7F10.6)
103 FORMAT(1HL,56H SELF-SHIELDING FACTORS USED ARE (HIGH TO LOW ENE
1RGY)/)
104 FORMAT(1HK,14H VULCAN ID = ,I2,2X,9F10.6/(15X,9F10.6))
      IF(NTM.GT.1) RETURN
      WRITE(6,103)
      READ(5,100) NSSF
      READ(5,100) (JSMAT(I),I=1,NSSF)
200 DO 201 ISF=1,NSSF
201 READ(5,101) (SST(ISF,J),J=1,NG)
500 DO 502 NSF=1,NSSF
      ISM=JSMAT(NSF)
      DO 503 MIG=1,NG
      ABSIG(ISM,MIG)=ABSIG(ISM,MIG)*SST(NSF,MIG)
503 SANSNU(ISM,MIG)=SANSNU(ISM,MIG)*SST(NSF,MIG)
      WRITE(6,104) JSMAT(NSF),(SST(NSF,J),J=1,NG)
502 CONTINUE
      RETURN
      END

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SIRFTC BURNT DECK
SUBROUTINE BURNUP
C
C
C PERFORMS ACTUAL DEPLETION, POWER, CONVERSION RATIO, AND
C FUEL INVENTORY CALCULATIONS
C
COMMON/SET1/ ATDEN(190,40),CONRAT(200),TOPOW(200),VOL(200),RR(200)
1,RATBEF(200)
COMMON/SET2/ FLUX(3800),SANSNU(40,20),ABSIG(40,20),SST(20,20),
1 XNU(12,20)
COMMON/SET3/ NMIR(200),RAUM(200),KAK(200),DELK(200),MIDKZ(400),
INEMO(200)
COMMON/SET4/ ATWGT(20),DECAY(40),THERNU(40),TNAME(40),FRACT(10),
1NRPT(40),IDVULC(40)
COMMON/SET5/ VULABS(40),VULFIS(40),VULAMF(40),ADPREV(40),IDYLD(10)
1,Y(20,12),KRON(20)
COMMON/SET6/ NG,NM,NISOT,TOTVOL,FIWATT,POWER,AVPOW,FNDRM
COMMON/SET7/ NFIS,NSPMAT,NBPOI,NFRACT,NYDNUC,NFIRPT
COMMON/SET8/ DELHR,KHAIN,AVOGAD,NZONR,MURDUT,MOOD,DIFFER,NONDPL
COMMON/SET9/ KTIM,NTINC,KRELL,KCELL
COMMON/SET10/ KOMP,KONG,KOG,LAND,DELSEC,MESH
COMMON/SET11/ NSSF,JSMAT(20)
DIMENSION PIM(40),FADINI(40),BEGMAS(12),FINMAS(12),POISON(40)
DIMENSION XNUMER(10),GAMSP(20), TFRAP(40),TAX(40)
DIMENSION PREDEN(40),PIZEN(40),VOLL(40)
C
C
C INITIALIZATION
C
AVGXE=0.0
AVGSM=0.0
AVGIO=0.0
AVGPM=0.0
DO 315 JAU=1,NISOT
TFRAP(JAU)=0.0
PREDEN(JAU)=0.0
PIZEN(JAU)=0.0
315 VOLL(JAU)=0.0
DO 50 MESH=1,NM
RATBEF(MESH)=0.0
CONRAT(MESH)=0.0
TOPOW(MESH)=0.0
50 CONTINUE
MESH=1
PETROL=0.0
IGLOO=0
POUR=0.0
DO 2 L=1,12
BEGMAS(L)=0.0
2 FINMAS(L)=0.0
BMASS=0.0
FMASS=0.0
TCONSB=0.0
IDESTB=0.0
TRBEF=0.0
MUF=0
KOOL=KOOL+1
DELSEC=3600.*DELHR
SECOND=DELSEC
C
C
C BEGINNING OF MESH ITERATION
C
1 CONTINUE
KONG=0
KUG=1
KUMP=1
LAND=0
KLON=0
JALP=0
JUME=0
11 DO 13 IDU=1,NISOT
ID=IDVULC(IDU)
VULABS(ID)=0.0
VULFIS(ID)=0.0
VULAMF(ID)=0.0
DO 14 IG=1,NG
LUC=NM*(IG-1)+MESH
VULABS(ID)=VULABS(ID)+ABSIG(ID,IG)*FLUX(LUC)
14 VULFIS(ID)=VULFIS(ID)+FLUX(LUC)*SANSNU(ID,IG)
13 VULAMF(ID)=VULABS(ID)-VULFIS(ID)
DENOM=0.0
DO 3 ID=1,12
IF(ATDEN(MESH,ID).NE.0.) GO TO 4
3 CONTINUE
DO 365 KURAN=1,NISOT
ID=IDVULC(KURAN)
IF(ATDEN(MESH,ID).NE.0..AND.NRPT(KURAN).NE.0) GO TO 4
365 CONTINUE
IF(NBPOI.EQ.0) GO TO 380
JALP=NSPMAT+18
JUME=JALP+NBPOI-1
DO 381 KOCH=JALP,JUME
IF(ATDEN(MESH,KOCH).EQ.0.) GO TO 381
KLUN=1 INDICATES THAT A BURNABLE POISON WILL BE CONSIDERED
IN THE CURRENT MESH
KLUN=1
GO TO 35
381 CONTINUE
380 MSUM=0
DO 10 KAT=1,NZONR

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      MSUM=MSUM+MMIK(KAT)
      IF(MSUM.EQ.(MESH-1)) GO TO 35
10  CONTINUE
      WRITE(6,151) MSUM,MESH
      GO TO 35
4   CONTINUE
      IF(NBPOI.EQ.0) GO TO 19
      JAA=NSPMAT+18
      JZZ=JAA+NBPOI-1
      DO 413 LOOK=JAA,JZZ
      IF(ATDEN(MESH,LOOK).EQ.0.) GO TO 413
      KLON=1
      GO TO 19
413 CONTINUE
19  IF(KHAIN.GT.1) CALL FISEQ2(59)
      CALL FISEQ1(59)
C
C      FISSIION YIELDS (GAMMA)
C
9   DO 21 JO=1,NFIS
      ID=IDYLD(JQ)
      IF(NFIRPT.EQ.0.OR.ADPREV(ID).NE.0.) GO TO 361
      DO 360 KAU=1,NISQT
      IF((NRP(T(KAD)-10).NE.ID) GO TO 360
      KONE=IDVULC(KAU)
      XNUMER(JQ)=(ADPREV(KONE)+ATDEN(MESH,KONE))*VULFIS(KONE)/2.
      GO TO 21
360 CONTINUE
361 XNUMER(JQ)=(ADPREV(ID)+ATDEN(MESH,ID))*VULFIS(ID)/2.
      DENUM=DENUM+XNUMER(JQ)
C
C      DENUM=TOTAL FISSION RATE PER BARN-CM FOR ISOTOPES WITH
C      CONSIDERED YIELDS
C
      IF(DENUM.EQ.0.) GO TO 35
      GAMFP=0.0
      GAMXE=0.0
      GAMSMM=0.0
      GAMIOD=0.0
      GAMPPM=0.0
      DO 26 JIG=1,NSPMAT
26  GAMSPP(JIG)=0.0
      DO 20 IO=1,NFIS
      KU=IDYLD(IO)
      GAMXE=GAMXE+XNUMER(IO)*Y(2,KU)/DENUM
      GAMSMM=GAMSMM+XNUMER(IO)*Y(3,KU)/DENUM
      GAMIOD=GAMIOD+XNUMER(IO)*Y(4,KU)/DENUM
      GAMPPM=GAMPPM+XNUMER(IO)*Y(5,KU)/DENUM
      IF(NSPMAT.EQ.0) GO TO 20
      DO 27 JIG=1,NSPMAT
      JOG=JIG+5
27  GAMSPP(JIG)=GAMSPP(JIG)+XNUMER(IO)*Y(JOG,KU)/DENUM
20  GAMFP=GAMFP+XNUMER(IO)*Y(1,KU)/DENUM
      GAINXE=GAMXE*DENUM
      GAINIO=GAMIOD*DENUM
      GAINSM=GAMSMM*DENUM
      GAINPM=GAMPPM*DENUM
C
C      FISSION PRODUCT AGGREGATE (OR INDIVIDUAL PRODUCT)
C
      ADPREV(13)=ATDEN(MESH,13)
      ATDEN(MESH,13)=(GAMFP*DENUM/VULABS(13))*(1.-EXP(-DELSEC*VULABS(13)
1) )+ADPREV(13)*EXP(-DELSEC*VULABS(13))
      ITEM=0
      IF(MOROUT.LT.2) GO TO 44
      WRITE(6,107)
      WRITE(6,112) MESH
      WRITE(6,110)
44  IF(NFRACT.EQ.0) GO TO 34
3/  ITEM=ITEM+1
      IF(ITEM.GT.1) GO TO 40
      DELSFC=3600.*DELHR*FRACT(ITEM)
      GO TO 41
40  DELSEC=3600.*DELHR*(FRACT(ITEM)-FRACT(ITEM-1))
41  IF(ITEM.GT.1) GO TO 34
      SAVXE=ATDEN(MESH,14)
      SAVIO=ATDEN(MESH,16)
      SAVSM=ATDEN(MESH,15)
      SAVPM=ATDEN(MESH,17)
      IF(MOROUT.LT.2) GO TO 34
      WRITE(6,102)
C
C      EXPLICIT YIELD AND DECAY POISONS
C
C      PM149
C
34  ADPREV(17)=ATDEN(MESH,17)
      GAMTOS=0.0
      GAMTOX=0.0
      ATDEN(MESH,17)=ADPREV(17)*EXP(-DELSEC*(DECAY(17)+VULABS(17)))
1+ (GAINPM/(DECAY(17)+VULABS(17)))*(1.-EXP(-DELSEC*(DECAY(17)+
2VULABS(17))))
C
C      SM149
C
      ADPREV(15)=ATDEN(MESH,15)
      IF(ATDEN(MESH,17).NE.0.) GO TO 500
      GAMTOS=GAINPM+GAINSM
      ATDEN(MESH,15)=(ADPREV(15)*EXP(-DELSEC*VULABS(15))+GAMTOS*
1(1.-EXP(-DELSEC*VULABS(15))))/VULABS(15)
      GO TO 520

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500 ATDEN(MESH,15)= DECAY(17)* (ADPREV(17)*(DECAY(17)+VULABS(17))-
1GAINPM)/((DECAY(17)+VULABS(17))*(VULABS(15)-DECAY(17)-VULABS(17)))
2 *(EXP(-DELSEC*(DECAY(17)+VULABS(17)))-EXP(-DELSEC*(VULABS(15))))
3 +(GAINSM*(DECAY(17)+VULABS(17))+GAINPM*DECAY(17))/
4 ((DECAY(17)+VULABS(17))
5 *VULABS(15))*(1.-EXP(-DELSEC*(VULABS(15))))+ ADPREV(15)*
6 EXP(-DELSEC*(VULABS(15)))
C
C I135
C
520 ADPREV(16)=ATDEN(MESH,16)
ATDEN(MESH,16)=ADPREV(16)*EXP(-DELSEC*(DECAY(40)+VULABS(16)))
1 +(GAINIO/(DECAY(40)+VULABS(16)))*(1.-EXP(-DELSEC*(DECAY(40)
2 +VULABS(16))))
C
C XE135
C
30 ADPREV(14)=ATDEN(MESH,14)
IF(ATDEN(MESH,16).NE.0.) GO TO 510
GAMTOX=GAINIO+GAINXE
ATDEN(MESH,14)=(ADPREV(14)*EXP(-DELSEC*(DECAY(14)+VULABS(14)))+
1GAMTOX*(1.-EXP(-DELSEC*(DECAY(14)+VULABS(14))))/(DECAY(14)+VULABS
2(14))
GO TO 530
510 ATDEN(MESH,14)=DECAY(16)*(ADPREV(16)*(DECAY(16)+VULABS(16))-GAINIO
1)/(DECAY(16)+VULABS(16))*(DECAY(14)+VULABS(14) -DECAY(16)
2-VULABS(16)))+(EXP(-DELSEC*(DECAY(16)+VULABS(16)))-EXP(-DELSEC
3*(DECAY(14)+VULABS(14)))+(GAINXE*(DECAY(16)+VULABS(16))+GAINIO*
4 DECAY(16))/(DECAY(16)+VULABS(16)))*
5(DECAY(14)+VULABS(14)))*(1.-EXP(-DELSEC*(DECAY(14)+VULABS(14)
6)))+(ADPREV(14)*EXP(-DELSEC*(DECAY(14)+VULABS(14))))
530 IF(DELSEC.EQ.SECOND) GO TO 38
IF(NFRACT.EQ.0) GO TO 38
IF(MOROUT.LT.2) GO TO 300
WRITE(6,101) FRAC(TITEM),ATDEN(MESH,14),ATDEN(MESH,15),ATDEN(MESH,
116),ATDEN(MESH,17)
300 IF(ITEM.LT.NFRAC) GO TO 37
DELSEC=SECOND
ATDEN(MESH,14)=SAVXE
ATDEN(MESH,15)=SAVSM
ATDEN(MESH,16)=SAVIO
ATDEN(MESH,17)=SAVPM
GO TO 34
38 CALL XENON (GAINXE,GAINSM,GAINIO,GAINPM,AVGXE,AVGSM,AVGIO,AVGPM)
GO TO 39
C
C SPECIAL MATERIALS INCLUDING FISSION PRODUCT AGGREGATES
C
35 DO 42 NOD=1,17
42 ADPREV(NOD)=0.0
IF(KLON.EQ.0) GO TO 382
WRITE(6,107)
WRITE(6,112) MESH
WRITE(6,164)
GO TO 39
382 LUSH=MESH+NMIR(KAT+1)-1
IF(MOROUT.LT.2) GO TO 39
WRITE(6,107)
IF(NMIR(KAT+1).EQ.1) GO TO 43
WRITE(6,152) MESH,LUSH
WRITE(6,153) MESH,LUSH
GO TO 39
43 WRITE(6,112) MESH
WRITE(6,100) MESH
39 DO 362 KOT=1,NISOT
JOT=IDVULC(KOT)
IF(JOT.LE.17) GO TO 362
IF(NRPT(KOT).NE.0.AND.ATDEN(MESH,JOT).NE.0.) GO TO 362
ADPREV(JOT)=ATDEN(MESH,JOT)
362 CONTINUE
IF(DENOM.EQ.0.) GO TO 33
IF(NSPMAT.EQ.0) GO TO 33
DO 31 IV=1,NSPMAT
DACRON=0.0
DACRON=KRON(IV)
NON=IV+17
ADPREV(NON)=ATDEN(MESH,NON)
ATDEN(MESH,NON)=((GAMSP(IV)*DENOM+((ATDEN(MESH,NON-1)+
1 ADPREV(NON-1))*VULABS(NON-1)/2.)* DACRON) /VULABS(NON))*
2(1.-EXP(-DELSEC*(VULABS(NON)))+(ADPREV(NON)*EXP(-DELSEC*(VULABS(NON))
31 CONTINUE
C
C KRON(I) IS THE KRONECKER DELTA--EQUAL TO 1 IF SPECIAL MATERIAL(I)
C IS A DECAY PRODUCT OF THE SPECIAL MATERIAL(I-1) AND EQUAL TO
C ZERO OTHERWISE
C
33 DO 96 ISEU=1,NISOT
POISON(ISEU)=0.0
96 FPIM(ISEU)=0.0
IF(NBPOI.EQ.0) GO TO 48
IF(KLON.EQ.0) GO TO 48
403 DO 32 LUP=1,NBPOI
LOOP=LUP+NSPMAT+17
ADPREV(LUP)=ATDEN(MESH,LUP)
ATDEN(MESH,LUP)=ADPREV(LUP)*EXP(-DELSEC*(VULABS(LUP)))
32 CONTINUE
48 IF(DENOM.EQ.0.) GO TO 90

```

```

C
C
C
CONVERSION RATE AT THE START OF THE TIME INTERVAL

CONSHE=0.0
DESTBE=0.0
CONSBE=ADPREV(1)*VULAMF(1)+ADPREV(7)*VULAMF(7)+ADPREV(4)*VULAMF(4)
+ADPREV(10)*VULAMF(10)
DESTBE=ADPREV(3)*VULABS(3)+ADPREV(5)*VULABS(5)+ADPREV(9)*VULABS(9)
+ADPREV(11)*VULABS(11)+ADPREV(2)*VULABS(2)*DECAY(2)/(DECAY(2)
+VULABS(2))+ADPREV(8)*VULABS(8)*DECAY(8)/(DECAY(8)+VULABS(8))
IF(NFIRPT.EQ.0) GO TO 340
DO 341 NWT=1,NISOT
IF(NRPT(NWT).EQ.0) GO TO 341
IGT=IDVULC(NWT)
KROG=NRPT(NWT)-10
GO TO (342,344,343,342,343,343,342,344,343,342,343,343),KROG
342 CONSRE=CONSBE+ADPREV(IGT)*VULAMF(IGT)
GO TO 341
343 DESTBE=DESTBE+ADPREV(IGT)*VULABS(IGT)
GO TO 341
344 DESTBE=DESTBE+ADPREV(IGT)*VULABS(IGT)*DECAY(KROG)/(DECAY(KROG)+VUL
ABS(IGT))
341 CONTINUE
340 TCUNSB=TCUNSB+CONSBE*VOL(MESH)
TDESTB=TDESTB+DESTBE*VOL(MESH)
TRBEF=TCUNSB/TDESTB
81 IF(DESTBE.EQ.0.) GO TO 93
KATBEF(MESH)=CONSRE/DESTBE
93 CONTINUE

C
C
C
FUEL INVENTORY CALCULATION

DO 15 JINX=1,12
HEGMAS(JINX)=BEGMAS(JINX)+VOL(MESH)*ADPREV(JINX)*ATWGT(JINX)/
1 (AVOGAD*10.**(-21))
15 FINMAS(JINX)=FINMAS(JINX)+VOL(MESH)*ATDEN(MESH,JINX)*ATWGT(JINX)
1 (AVOGAD*10.**(-21))
FUEL=0.0
DO 71 ID=1,12
71 FUEL=FUEL+ATDEN(MESH,ID)*VULABS(ID)*VOL(MESH)
PETROL=PETROL+FUEL
IF(NFIRPT.EQ.0) GO TO 51
DO 49 MIT=1,NISOT
IF(NRPT(MIT).LE.0) GO TO 49
META=IDVULC(MIT)
MUTT=NRPT(MIT)-10
FUEL=FUEL+ATDEN(MESH,META)*VULABS(META)*VOL(MESH)
HEGMAS(MUTT)=BEGMAS(MUTT)+VOL(MESH)*ADPREV(META)*ATWGT(MUTT)/
1 (AVOGAD*10.**(-21))
FINMAS(MUTT)=FINMAS(MUTT)+VOL(MESH)*ATDEN(MESH,META)*ATWGT(MUTT)/
1 (AVOGAD*10.**(-21))
49 CONTINUE
PETROL=PETROL+FUEL

C
C
C
POISON CALCULATION

51 PHIDT=0.0
DO 72 ID=1,NISOT
IT=IDVULC(ID)
PIZEN(ID)=PIZEN(ID)+ATDEN(MESH,IT)*VULABS(IT)*VOL(MESH)
72 POISON(ID)=ATDEN(MESH,IT)*VULABS(IT)*VOL(MESH)/FUEL

C
C
C
POWER CALCULATION

75 CONTINUE
DO 92 ID=1,NISOT
ID0=IDVULC(ID)
IF(ID0.GT.12.AND.NRPT(ID).LE.0) GO TO 92
78 SUM=0.0
DO 91 IG=1,NG
LOC=NM*(IG-1)+MESH
41 SUM=SUM+SANSNU(ID0,IG)*FLUX(LOC)
FPIM(ID)=ATDEN(MESH,ID0)*10.**(-24)*SUM*VOL(MESH)/FIWATT
TOPOW(MESH)=TOPOW(MESH)+FPIM(ID)
92 CONTINUE
77 IF(TOPOW(MESH).EQ.0.) GO TO 90
IGL00=IGL00+1
DO 89 IOTA=1,NISOT
TFRAP(IOTA)=TFRAP(IOTA)+FPIM(IOTA)
89 FPIM(IOTA)=FPIM(IOTA)/TOPOW(MESH)
90 CONTINUE
NREG=17+NPMAT+NBPOI+NFIRPT+1
82 CONTINUE
DO 94 JSEQ=1,NISOT
FADIN(JSEQ)=0.0
IDT=IDVULC(JSEQ)
IF(ADPREV(IDT).EQ.0.) GO TO 94
FADIN(JSEQ)=ATDEN(MESH,IDT)/ADPREV(IDT)
PRDEN(JSEQ)=ADPREV(IDT)*VOL(MESH)+PRDEN(JSEQ)
94 CONTINUE
IF(MOROUT.LT.2) GO TO 47
45 WRITE(6,129)
79 CONTINUE
DO 95 JSEQ=1,NISOT
ID=IDVULC(JSEQ)
95 WRITE(6,120) TNAME(ID),JSEQ,ADPREV(ID),ATDEN(MESH,ID),FADIN(JSEQ),
1 FPIM(JSEQ),POISON(JSEQ)
47 POUW=POUW+TOPOW(MESH)
IF(DENOM.GT.0.0R.KLON.EQ.1) GO TO 320
MOST=MESH+1
DO 321 LITE=MOST,LUSH

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DO 322 NITE=1,NISOT
MITE=IDVULC(NITE)
322 PREDEN(NITE)=ADPREV(MITE)*VOL(LITE)+PREDEN(NITE)
321 CONTINUE
MESH=LUSH
320 MESH=MESH+1
IF(MESH.LE.NM) GO TO 1
POWIR=POWER
NONE=KTIM-1
IF(NONE.GT.1) WRITE(6,107)
WRITE(6,156) NONE
CALL ABSPHI

C
C PRODUCTION RATE OF FISSIONABLE MATERIAL
C USING NEW FLUX LEVEL AT END OF INTERVAL
C
SPACE=0.0
DO 87 MESH=1,NM
TOPOW(MESH)=TOPOW(MESH)*FNORM
IF(TOPOW(MESH).NE.0.) SPACE=SPACE+VOL(MESH)
DO 85 IOTA=1,NISOT
ID=IDVULC(IOTA)
VULABS(ID)=0.0
VULFIS(ID)=0.0
VULAMF(ID)=0.0
DO 86 IG=1,NG
LOC=NM*(IG-1)+MESH
VULABS(ID)=VULABS(ID)+ABSIG(ID,IG)*FLUX(LOC)
86 VULFIS(ID)=VULFIS(ID)+FLUX(LOC)*SANSNU(ID,IG)
85 VULAMF(ID)=VULABS(ID)-VULFIS(ID)
PROD=0.0
PROD = ATDEN(MESH,1)*VULAMF(1)+ATDEN(MESH,7)*VULAMF(7)+
1 ATDEN(MESH,4)*VULAMF(4)+ATDEN(MESH,10)*VULAMF(10)

C
C DESTRUCTION RATE OF FISSIONABLE MATERIAL
C
DEST=0.0
DEST = ATDEN(MESH,3)*VULABS(3)+ATDEN(MESH,5)*VULABS(5)
1 + ATDEN(MESH,9)*VULABS(9)+ATDEN(MESH,11)*VULABS(11)+ATDEN(MESH,2)
2 *VULABS(2)*DECAY(2)/(DECAY(2)+VULABS(2))+ATDEN(MESH,8)*
3 VULABS(8)*DECAY(8)/(DECAY(8)+VULABS(8))
IF(NFIRPT.EQ.0) GO TO 350
DO 351 NMT=1,NISOT
IF(NRPT(NMT).EQ.0) GO TO 351
IGT=IDVULC(NMT)
KROG=NRPT(NMT)-10
GO TO (352,354,353,352,353,352,354,353,352,353,353),KROG
352 PROD=PROD+ATDEN(MESH,IGT)*VULAMF(IGT)
GO TO 351
353 DEST=DEST+ATDEN(MESH,IGT)*VULABS(IGT)
GO TO 351
354 DEST=DEST+ATDEN(MESH,IGT)*VULABS(IGT)*DECAY(KROG)/(DECAY(KROG)+
1 VULABS(IGT))
351 CONTINUE
350 TPROD=TPROD+PROD*VOL(MESH)
TDEST=TDEST+DEST*VOL(MESH)
IF(DEST.EQ.0.) GO TO 97

C
C CONVERSION RATE OF FISSIONABLE MATERIAL
C
CONRAT(MESH)=PROD/DEST
GO TO 98
97 CONRAT(MESH)=0.0
98 TCONV=0.0
IF(TDEST.EQ.0.) GO TO 87
TCONV=TPROD/TDEST
87 CONTINUE
AVGXE=AVGXE/SPACE
AVGSM=AVGSM/SPACE
AVGIO=AVGIO/SPACE
AVGPM=AVGPM/SPACE
FIGLOD=IGLOD
IF(NFIRPT.LE.0) GO TO 370
DO 371 JAR=1,NISOT
IF(NRPT(JAR).LE.0) GO TO 371
MATTE=NRPT(JAR)-10
DO 410 LOCH=1,NISOT
IF(MATTE.EQ.IDVULC(LOCH)) GO TO 411
410 CONTINUE
411 TFRAP(LOCH)=TFRAP(LOCH)+TFRAP(JAR)
371 CONTINUE
370 DO 83 JAM=1,NISOT
83 TFRAP(JAM)=TFRAP(JAM)/POUR
DO 73 ID=1,NISOT
73 TAX(ID)=0.0
DO 74 MESH=1,NM
DO 74 KEN=1,NISOT
ID=IDVULC(KEN)
IF(ATDEN(MESH,ID).NE.0.) VOLL(KEN)=VOLL(KEN)+VOL(MESH)
74 TAX(KEN)=TAX(KEN)+ATDEN(MESH,ID)*VOL(MESH)
KTL=KTIM-1
WRITE(6,105) KTL
WRITE(6,121)
DO 80 N0ID=1,NISOT
ID=IDVULC(N0ID)
WRITE(6,104) ID
WRITE(6,106) (ATDEN(IMESH,ID),IMESH=1,NM),TAX(N0ID)
80 CONTINUE
WRITE(6,113) KTL,(RATBEF(ION),ION=1,NM)
WRITE(6,114) KTL,(CONRAT(ION),ION=1,NM)
WRITE(6,109)

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      WRITE(6,117)
      DO 550 JMESH=1,NM
550  TOPOW(JMESH)=TOPOW(JMESH)/VOL(JMESH)
      WRITE(6,106) (TOPOW(JMESH),JMESH=1,NM),POWER
      DO 88 JERK=1,NM
88  TOPOW(JERK)=TOPOW(JERK)/AVPOW
      WRITE(6,117)
      WRITE(6,115)
      WRITE(6,117)
      WRITE(6,106) (TOPOW(JMESH),JMESH=1,NM),AVPOW
      WRITE(6,110)
      WRITE(6,119) POWER
      WRITE(6,118) AVGXE,AVGSM,AVGIO,AVGPM
      WRITE(6,111)
      DO 330 ID=1,NISOT
      NOW=IDVULC(ID)
      IF(NOW.GT.12) GO TO 330
      WRITE(6,158) TNAME(NOW),BEGMAS(NOW),FINMAS(NOW),TFRAP(ID)
330  CONTINUE
      DO 301 ID=1,NISOT
      PIZEN(ID)=PIZEN(ID)/PETROL
      PREDEN(ID)=PREDEN(ID)/VOLL(ID)
301  TAX(ID)=TAX(ID)/VOLL(ID)
      WRITE(6,107)
      WRITE(6,157)
      WRITE(6,155)
311  CONTINUE
      DO 310 ID=1,NISOT
      JSEQ=IDVULC(ID)
312  IF(PREDEN(ID).EQ.0.) GO TO 313
      XSU=TAX(ID)/PREDEN(ID)
      GO TO 310
313  XSU=0.0
310  WRITE(6,120) TNAME(JSEQ),JSEQ,PREDEN(ID),TAX(ID),XSU,PIZEN(ID)
      DO 84 LAWN=1,12
      BMASS=BMASS+BEGMAS(LAWN)
84  FMASS=FMASS+FINMAS(LAWN)
      BEGLB=2.2046*BMASS
      FINLB=2.2046*FMASS
      DEGBUR=2200.*POWER*DELHR/(24.*(BEGLB-FINLB))*10.**(-6)
      WRITE(6,116)
      WRITE(6,140) BMASS,FMASS
      WRITE(6,141) BEGLB,FINLB
      WRITE(6,108) TRBEF,TCNV
      WRITE(6,103) POWIN,POUR
      WRITE(6,154) DEGBUR
C
100  FORMAT(1HL,80H NO FISSIONABLE ISOTOPES OR CONSEQUENT FISSION PROD
      UCTS OCCUR IN MESH INTERVAL ,I4)
101  FORMAT(F9.2,5X,E15.6)
102  FORMAT(71H FISSION PRODUCT CONCENTRATIONS AT VARIOUS FRACTIONS O
      IF THE TIME STEP//12H FRACTION,9X,5HXE135,10X,5HSM149,10X,5HI 13
      25,10X,5HPM149)
103  FORMAT(1HK,24H TOTAL POWER IN WATTS,10X,E12.6,10X,E12.6)
104  FORMAT(1HL,15H VULCAN ID = ,I2)
105  FORMAT(1HL,84H NEW ATOM DENSITIES BY MESH INTERVAL FOR EACH ISOT
      OPE AT THE END OF TIME INTERVAL ,I2)
106  FORMAT(8E14.6)
107  FORMAT(1HL)
108  FORMAT(1HK,24H CONVERSION RATIO ,10X,E12.6,10X,E12.6)
109  FORMAT(1HK,34H BY MESH INTERVAL THE POWER IS )
110  FORMAT(1HK)
111  FORMAT(1HL,64H FISSIONABLE INITIAL MASS IN FINAL MASS IN
      1 FRACTIONAL/6X,7HISOTOPE,9X,9HKILOGRAMS,9X,9HKILOGRAMS,9X,5HPPOWER
      2/)
112  FORMAT(10(3H **),22H MESH INTERVAL NUMBER ,I4,1X,10(3H **))
113  FORMAT(1HL,63H CONVERSION RATIO BY MESH INTERVAL AT START OF TI
      ME INTERVAL,I2//(8E14.6))
114  FORMAT(1HK,63H CONVERSION RATIO BY MESH INTERVAL AT END OF TI
      ME INTERVAL,I2//(8E14.6))
115  FORMAT(73H BY MESH INTERVAL THE POWER (NOKMALIZED TO VOLUME-AV
      ERAGED POWER) IS )
116  FORMAT(1HL,37X,7HINITIAL,16X,5HFINAL)
117  FORMAT(1HJ)
118  FORMAT(1HL,14X,14HXENON-135 ,E20.6//15X,14HSAMARIUM-149 ,E20.
      6//15X,14HIODINE-135 ,E20.6//15X,14HPROMETHIUM-149,E20.6)
119  FORMAT(75H VOLUME AVERAGED EQUILIBRIUM CONCENTRATIONS OF MAJOR
      1 FISSION PRODUCTS /45H IN NUCLEI/BARN*CM IF THE POWER LEVEL O-
      2F ,E11.6,20H WATTS IS MAINTAINED)
120  FORMAT(3X,A5,1H-,I2,1X,6E15.6)
121  FORMAT(1HK,20(4H****)//103H LAST ENTRY FOR EACH ISOTOPE
      1 IS ONE BARN-TH OF ITS TOTAL NUMBER OF ATOMS IN THE CONFIGURATION//
      21X,20(4H****))
129  FORMAT(1HK,9H ISOTOPE,10X,4HN(T),9X,6HN(T+1),7X,11HN(T+1)/N(T),7X
      1,5HPOWER,9X,6HPOISON/10H DESCR.,53X,8HFRACTION,7X,6HFACTOR)
131  FORMAT(1HK,26H IN MESH INTERVAL NUMBER ,I4//15X,58HTHE CONVERSION
      1 RATIO AT THE START OF THE TIME INTERVAL IS ,F10.6//15X,58HTHE CON
      2VERSION RATIO AT THE END OF THE TIME INTERVAL IS ,F10.6)
140  FORMAT(1HL,24H TOTAL FUEL INVENTORY,10X,E12.6,10X,E12.6/9X,
      112HIN KILOGRAMS)

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141 FORMAT(1HK,24H      TOTAL FUEL INVENTORY,10X,E12.6,10X,E12.6/10X,
19HIN POUNDS)
150 FORMAT(7E15.6)
151 FORMAT(1HL,10(1H+),95H  ALL ZONES HAVE BEEN SEARCHED FOR A NON-DEP
1LETABLE ZONE STARTING POINT AND NONE HAS BEEN FOUND/1X,10(1H+),
29H  MSUM = ,I3,12H AND MESH = ,I3)
152 FORMAT(10(3H **),23H MESH INTERVALS NUMBER ,I4,6H THRU ,I4,
11X,10(3H **))
153 FORMAT(1HL,81H  NO FISSIONABLE ISOTOPES OR CONSEQUENT FISSION PROD
UCTS OCCUR IN MESH INTERVALS ,I4,6H THRU ,I4)
154 FORMAT(1HK,28H      MEGAWATT-DAYS/METRIC TON,1/X,E12.6)
155 FORMAT(1HK,9H  ISOTOPE,10X,4HN(T),9X,6HN(T+1),7X,11HN(T+1)/N(T),7X
1,6HPOISON/10H  DESCR.,54X,6HFACTOR)
156 FORMAT(1HL,60H  ALL MESH INTERVALS HAVE BEEN TRAVERSED FOR TIME I
INTERVAL ,I2/64H  THE FLUX WILL NOW BE RENORMALIZED FOR THE NEXT
2TIME INTERVAL)
157 FORMAT(1HK,84H THE FOLLOWING EDIT IS AVERAGED OVER ALL MESH INTERV
IALS IN WHICH EACH ISOTOPE OCCURS)
158 FORMAT(1HK,6X,A5,E20.6,E18.6,E16.6)
164 FORMAT(1HL,56H  CONTAINS BURNABLE POISONS BUT NO FISSIONABLE ISOT
OPES)
      RETURN
      END

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$IBFTC FADE DECK
SUBROUTINE FISEQ1(*)
C
C
C SOLVES DEPLETION EQUATIONS FOR FISSIONABLE ISOTOPES
C
COMMON/SET1/ ATDEN(190,40),CONRAT(200),TOPOW(200),VOL(200),RR(200)
1,RATBEF(200)
COMMON/SET2/ FLUX(3800),SANSNU(40,20),ABSIG(40,20),SST(20,20),
1 XNU(12,20)
COMMON/SET3/ NMIR(200),RAUM(200),RAR(200),DELR(200),MIDRZ(400),
1NEMO(200)
COMMON/SET4/ATWGT(20),DECAY(40),THERNU(40),TNAME(40),FRACT(10),
1NRPT(40),IDVULC(40)
COMMON/SET5/ VULABS(40),VULFIS(40),VULAMF(40),ADPREV(40),IDYLD(10)
1,Y(20,12),KRON(20)
COMMON/SET6/ NG,NM,NISOT,TOTVOL,FIWATT,POWER,AVPOW,FNORM
COMMON/SET7/NFIS,NSPMAT,NBPOI,NFRACT,NYDNUC,NFIRPT
COMMON/SET8/ DELHR,KHAIN,AVOGAD,NZONR,MOROUT,MOOD,DIFFER,NONDPL
COMMON/SET9/ KTIM,NTINC,KRELL,KCELL
COMMON/SET10/ KOMP,KONG,KOG,LAND,DELSEC,MESH
J1=1
J2=2
J3=3
J4=4
J5=5
J6=6
GO TO 201
ENTRY FISEQ2(*)
22 J1=7
J2=8
J3=9
J4=10
J5=11
J6=12
8 GO TO (201,202,203,204,205,206),KOMP
C
C ENTER THE ATOM DENSITY CALCULATIONS FOR FISSIONABLE ISOTOPES
C
C TH232 OR U238
C
201 ADPREV(J1)=ATDEN(MESH,J1)
ATDEN(MESH,J1)=ADPREV(J1)*(2.-VULABS(J1)*DELSEC)/(2.+VULABS(J1)*
1 DELSEC)
IF(LAND.EQ.1) GO TO 7
C
C PA233 OR NP239
C
202 ADPREV(J2)=ATDEN(MESH,J2)
ATDEN(MESH,J2)=(ATDEN(MESH,J1)+ADPREV(J1))*VULAMF(J1)/(2.*(
1 DECAY(J2)+VULABS(J2)))+(ADPREV(J2)-(ATDEN(MESH,J1)+ADPREV(J1))
2 VULAMF(J1)/(2.*(DECAY(J2)+VULABS(J2))))*EXP(-(DECAY(J2)+
3 VULABS(J2))*DELSEC)
IF(ATDEN(MESH,J2).EQ.0.) ATDEN(MESH,J2)=-ATDEN(MESH,J2)
IF(LAND.EQ.1) GO TO 7
C
C U233 OR PU239
C
203 ADPREV(J3)=ATDEN(MESH,J3)
DO 30 IKE=1,NISOT
NIKE=IDVULC(IKE)
IF(NIKE.NE.J2) GO TO 30
GO TO 32
30 CONTINUE
ATDEN(MESH,J3)=(ADPREV(J3)*(2.-VULABS(J3)*DELSEC)+(ATDEN(MESH,J1)
1 +ADPREV(J1))*VULAMF(J1)*DELSEC)/(2.+VULABS(J3)*DELSEC)
GO TO 31
32 ATDEN(MESH,J3)=(ADPREV(J3)*(2.-VULABS(J3)*DELSEC)+(ATDEN(MESH,J2)+
1 ADPREV(J2))*DECAY(J2)*DELSEC)/(2.+VULABS(J3)*DELSEC)
31 IF(LAND.EQ.1) GO TO 7
C
C U234 OR PU240
C
204 ADPREV(J4)=ATDEN(MESH,J4)
ATDEN(MESH,J4)=(ADPREV(J4)*(2.-VULABS(J4)*DELSEC)+(ATDEN(MESH,J3)+
1 ADPREV(J3))*VULAMF(J3)*DELSEC+(ATDEN(MESH,J2)+ADPREV(J2))*
2 VULAMF(J2)*DELSEC)/(2.+VULABS(J4)*DELSEC)
IF(LAND.EQ.1) GO TO 7
C
C U235 OR PU241
C
205 ADPREV(J5)=ATDEN(MESH,J5)
IF(J5.EQ.11) GO TO 29
IF(KONG.EQ.11) GO TO 29
U235
ATDEN(MESH,J5)=(ADPREV(J5)*(2.-VULABS(J5)*DELSEC)+(ATDEN(MESH,J4)
1 +ADPREV(J4))*VULAMF(J4)*DELSEC)/(2.+VULABS(J5)*DELSEC)
IF(LAND.EQ.1) GO TO 7
GO TO 206
PU241
29 ATDEN(MESH,J5)=(ADPREV(J5)*(2.-(VULABS(J5)+DECAY(11))*DELSEC)+
1 (ATDEN(MESH,J4)+ADPREV(J4))*VULAMF(J4)*DELSEC)/(2.+(VULABS(J5)+
2 DECAY(11))*DELSEC)
IF(LAND.EQ.1) GO TO 7

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C
C      U236 OR PU242
C
206 ADPREV(J6)=ATDEN(MESH,J6)
    ATDEN(MESH,J6)=(ADPREV(J6)*(2.-VULABS(J6)*DELSEC)+(ATDEN(MESH,J5)
    1+ADPREV(J5))*VULAMF(J5)*DELSEC)/(2.+VULABS(J6)*DELSEC)
    IF(LAND.EQ.1) GO TO 7
    IF(J6.EQ.12) GO TO 25
    IF(KHAIN.EQ.0) GO TO 22
C
C      FOR DUPLICATE FISSIONABLE ISOTOPIES
C
25 IF(INFIRPT.EQ.0) RETURN 1
    LAND=1
    7 IF(KOG.GT.NISOT) RETURN 1
    DO 5 KIM=KOG,NISOT
    IF(NRPT(KIM).EQ.0) GO TO 5
    JADE=IDVULC(KIM)
    IF(ATDEN(MESH,JADE).EQ.0.) GO TO 5
    KOG=KIM+1
    KOMP=NRPT(KIM)-10
    IF(KOMP.GT.6) GO TO 320
    J1=1
    J2=2
    J3=3
    J4=4
    J5=5
    J6=6
    GO TO 330
320 J1=7
    J2=8
    J3=9
    J4=10
    J5=11
    J6=12
330 GO TO (301,302,303,304,305,306,301,302,303,304,305,306),KOMP
301 J1=IDVULC(KIM)
    GO TO 315
302 J2=IDVULC(KIM)
    IF(NRPT(KIM-1).EQ.11) J1=IDVULC(KIM-1)
    IF(NRPT(KIM-1).EQ.17) J1=IDVULC(KIM-1)
    GO TO 315
303 J3=IDVULC(KIM)
    IF(NRPT(KIM-1).EQ.12) J2=IDVULC(KIM-1)
    IF(NRPT(KIM-1).EQ.18) J2=IDVULC(KIM-1)
    GO TO 315
304 J4=IDVULC(KIM)
    IF(NRPT(KIM-1).EQ.13) J3=IDVULC(KIM-1)
    IF(NRPT(KIM-1).EQ.19) J3=IDVULC(KIM-1)
    IF(NRPT(KIM-2).EQ.12) J2=IDVULC(KIM-2)
    IF(NRPT(KIM-2).EQ.18) J2=IDVULC(KIM-2)
    GO TO 315
305 J5=IDVULC(KIM)
    IF(NRPT(KIM-1).EQ.14) J4=IDVULC(KIM-1)
    IF(NRPT(KIM-1).EQ.20) J4=IDVULC(KIM-1)
    GO TO 315
306 J6=IDVULC(KIM)
    IF(NRPT(KIM-1).EQ.15) J5=IDVULC(KIM-1)
    IF(NRPT(KIM-1).EQ.21) J5=IDVULC(KIM-1)
315 CONTINUE
    KONG=KOMP
    IF(KOMP.LE.6) GO TO 8
    KOMP=KOMP-6
    GO TO 8
    5 CONTINUE
    RETURN
    END

```

```

SIBFTC POUT DECK
SUBROUTINE OUTPUT
COMMON/SET1/ ATDEN(190,40),CONRAT(200),TOPOW(200),VOL(200),RR(200)
1,KATBEF(200)
COMMON/SET2/ FLUX(3800),SANSNU(40,20),ABSIG(40,20),SST(20,20),
1 XNU(12,20)
COMMON/SET3/ NMIR(200),RAUM(200),RAR(200),DELK(200),MIDRZ(400),
1NEMO(200)
COMMON/SET4/ATWGT(20),DECAY(40),THERNU(40),TNAME(40),FRACT(10),
1NRPT(40),IDVULC(40)
COMMON/SET5/ VULABS(40),VULFIS(40),VULAMF(40),ADPREV(40),IDVLD(10)
1,Y(20,12),KRON(20)
COMMON/SET6/ NG,NM,NISUT,TOTVOL,FIWATT,POWER,AVPOW,FNORM
COMMON/SET8/ DELHK,KHAIN,AVOGAD,NZONR,MOROUT,MOOD,DIFFER,NONDPL
DIMENSION LABEL(40),CARRY(40)

C
C OUTPUT EDIT - DETERMINATION OF THE NUMBER OF MATERIALS TO BE
C NEEDED FOR THE NEXT TOSN CALC - CALCULATES AVERAGE ATOM DENSITIES
C AFTER CHOOSING ON THE BASIS OF MACROSCOPIC COMPARISONS
C
100 FORMAT(1H1)
102 FORMAT(1H1,46H THE ATOM DENSITIES TO BE USED FOR THE NEXT ,I2,
117H MESH INTERVALS ( ,I3,5H THRU ,I3, 5H) ARE//)
103 FORMAT(8E14,6)
104 FORMAT(2H= ,7I10)
105 FORMAT(7E10,5)
106 FORMAT(1H1,10X,23HMACG INPUT CARD IMAGES//)
107 FORMAT(5X,7E14,6)
108 FORMAT(1H1,88H MACROSCOPIC ABSORPTION CROSS SECTIONS BY MESH INT
1ERVAL,BY GROUP,AND BY VULCAN ISOTOPE)
109 FORMAT(1H1,24H MESH INTERVAL NUMBER ,I3)
110 FORMAT(1H1,23H ENERGY GROUP NUMBER ,I3//)
111 FORMAT(1H1,25H MESH INTERVALS NUMBER ,I3,6H THRU ,I3)
112 FORMAT(56H AVERAGED ATOM DENSITIES FOR NEXT SPATIAL CALCULATION
1)
151 FORMAT(1H1,10(1H+),95H ALL ZONES HAVE BEEN SEARCHED FOR A NON-DEP
1LETABLE ZONE STARTING POINT AND NONE HAS BEEN FOUND/1X,10(1H+),
29H MSUM = ,I3,12H AND MESH = ,I3)
LGO=0
LSTOP=0
IF(MOROUT.EQ.0.OR.MOROUT.EQ.2) GO TO 57

C
C THROUGH STATEMENT 42 IS THE MACROSCOPIC PRINTOUT
C
WRITE(6,108)
KOKO=NISUT-NONDPL
LIM=0
55 LIM=LIM+1
MSUM=0
DO 50 MID=1,KOKO
IF(ATDEN(LIM,MID).NE.0.) GO TO 52
50 CONTINUE
DO 51 KAT=1,NZONR
MSUM=MSUM+NMIR(KAT)
IF(MSUM.EQ.(LIM-1)) GO TO 53
51 CONTINUE
WRITE(6,151) MSUM,LIM
53 IF(NMIR(KAT+1).EQ.1) GO TO 52
LUSH=LIM+NMIR(KAT+1)-1
WRITE(6,111) LIM,LUSH
GO TO 54
52 WRITE(6,109) LIM
54 DO 43 LIG=1,NG
DO 44 NIB=1,NISUT
LID=IDVULC(NIB)
IF(ABSIG(LID,LIG).EQ.0.) ADPREV(LID)=ATDEN(LIM,LID)
44 ATDEN(LIM,LID)=ATDEN(LIM,LID)*ABSIG(LID,LIG)
WRITE(6,110) LIG
WRITE(6,103) (ATDEN(LIM,LID),LID=1,NISUT)
DO 45 NIB=1,NISUT
LID=IDVULC(NIB)
IF(ABSIG(LID,LIG).EQ.0.) GO TO 47
ADTEN(LIM,LID)=ADTEN(LIM,LID)/ABSIG(LID,LIG)
GO TO 45
47 ATDEN(LIM,LID)=ADPREV(LID)
45 CONTINUE
43 CONTINUE
IF(MSUM.NE.0) LIM=LUSH
42 IF(LIM.LT.NM) GO TO 55

C
C
57 MESH=0
IF(DIFFER.EQ.0.) DIFFER=.001
C THE FOLLOWING LOOP IS FOR THE ZONE SWEEP
5 DO 10 IX=1,NZONR
LGO=LSTOP+1
LSTOP=LGO+NMIR(IX)-1
MAR=0
METRO=0
MESH=MESH+1
JAVE=0
METRO=NMIR(IX)-1
IF(METRO.EQ.0) GO TO 17
C FOLLOWING IS THE MESH SWEEP WITHIN ZONE IX
DO 11 JX=1,METRO
KAVG=1
MESH=MESH+1
14 RAVG=KAVG
COVA=DIFFER*RAVG
MM=MESH-KAVG
LGRUP=0

```



```

      JIG=0
C     SWEEP BY GROUP
      1 JIG=JIG+1
C     THE 40 LOOP IS TO CHANGE TO MACROSCOPIC CROSS SECTIONS FOR
C     COMPARISON TEST BY MESH INTERVAL WITHIN REGION, THEN BY ISOTOPE
      DO 40 JIM=MM,MESH
      FAHR=0.0
      DO 40 NIB=1,NISOT
      JID=IDVULC(NIB)
      IF(ABSIG(JID,JIG).GT.0.) GO TO 56
      ATDEN(JIM,JID)=-ATDEN(JIM,JID)
      GO TO 40
      56 ATDEN(JIM,JID)=ATDEN(JIM,JID)*ABSIG(JID,JIG)
      IF(JIM.NE.MESH) GO TO 40
      IF(JID.LE.12.OR.NRPT(NIB).NE.0) FAHR=FAHR+ATDEN(MESH,JID)
      40 CONTINUE
      FAHR=10.*FAHR*DIFFER
C     THE 12 LOOP IS THE COMPARISON TEST BY ISOTOPE FOR GROUP JIG
      3 DO 12 KQZ=1,NISOT
      IDZ=IDVULC(KQZ)
      IF(ATDEN(MM,IDZ).GT.0.) GO TO 15
      QUOI=1.0
      GO TO 16
      15 QUOI=ATDEN(MESH,IDZ)/ATDEN(MM,IDZ)
      16 IF(ABS(QUOI-1.).GT.COVA.AND.ATDEN(MESH,IDZ).GT.FAHR) GO TO 19
      12 CONTINUE
      LGKUP=LGKUP+1
      GO TO 19
      4 IF(KAVG.GT.1) GO TO 13
C
C     JAVE AT ANY TIME IS EQUAL TO THE NUMBER OF ADJACENT COMPARISONS OR
C     MATCHES THAT HAVE ALREADY BEEN FOUND FOR ANY ONE AVERAGED SET.
C
      JAVE=JAVE+1
      IF(JAVE.GT.1) GO TO 13
C
C     KARY IS THE MESH INTERVAL THAT CONTAINS THE AVERAGED ATOM DENSITY
C     (NEGATIVE) FOR THE NEXT MESHM1-KARY MESH INTERVALS.
C
      KARY=MESH-1
      13 IF(MM.EQ.KARY) GO TO 18
C
C     KAVG IS THE ORDER OF THE COMPARISON AT HAND, E.G., =1 IF COMPAR-
C     ISON IS WITH AN ADJACENT MESH INTERVAL, =2 IF THE COMPARISON IS
C     WITH THE MESH INTERVAL SECOND PRECEDING THE CURRENT ONE, ETC.
C
      KAVG=KAVG+1
      GO TO 14
      18 IF(JX.EQ.METRD) GO TO 22
      GO TO 11
C     THE 46 LOOP CHANGES BACK TO ATOM DENSITIES FROM MACROSCOPICS
      19 DO 46 JIM=MM,MESH
      DO 46 NOK=1,NISOT
      JID=IDVULC(NOK)
      IF(ABSIG(JID,JIG).GT.0.) GO TO 90
      ATDEN(JIM,JID)=-ATDEN(JIM,JID)
      GO TO 46
      90 ATDEN(JIM,JID)=ATDEN(JIM,JID)/ABSIG(JID,JIG)
      46 CONTINUE
      IF(JIG.LT.NG) GO TO 1
      IF(LGKUP.EQ.NG) GO TO 4
      IF(KAVG.EQ.JAVE) GO TO 26
      IF(JAVE.NE.0) GO TO 26
      KARY=MESH-1
      MESHM1=KARY
      IF(JX.EQ.METRD) MARS=1
      GO TO 21
      26 MESHM1=MESH-1
      IF(JX.EQ.METRD) MARS=1
      GO TO 21
      22 MESHM1=MESH
      21 VOLUME=0.0
      DO 25 MED=KARY,MESHM1
      VOLUME=VOLUME+VOL(MED)
      DO 23 KDA=1,NISOT
      IDA=IDVULC(KDA)
      AXE=0.0
      DO 24 MED=KARY,MESHM1
      AXE=AXE+ATDEN(MED,IDA)*VOL(MED)
      23 ATDEN(KARY,IDA)=-AXE/VOLUME
      11 CONTINUE
      IF(MARS.EQ.1) GO TO 17
      GO TO 10
      17 DO 27 JMP=1,NISOT
      IMP=IDVULC(JMP)
      27 ATDEN(MESH,IMP)=-ATDEN(MESH,IMP)
      10 CONTINUE

```

```

C
C   CALCULATION OF MACGG INPUT
C
      WRITE(6,100)
      WRITE(6,112)
      NUDE=0
      MOOD=0
      DO 32 MOUT=1,NM
      DO 31 KOUT=1,NISOT
      ID=IDVULC(KOUT)
      IF(ATDEN(MOUT,IOUT).GT.0.) GO TO 30
      ATDEN(MOUT,IOUT)=-ATDEN(MOUT,IOUT)
31  CONTINUE
      IF(NUDE.EQ.0) GO TO 30
33  NEND=MOUT-1
      NART=MOUT-NUDE
      WRITE(6,102) NUDE,NART,NEND
      MOOD=MOOD+1
      NEMD(MOOD)=NUDE
      LUG=0
C
C   REARRANGE ATOM DENSITIES IN MACGG ORDER AND COMPRESS
C
      DO 36 MX=1,NISOT
      ID=IDVULC(MX)
      CARRY(MX)=0.0
36  CARRY(MX)=ATDEN(NART,ID)
      DO 35 LX=1,NISOT
      IF(CARRY(LX).LE.0.) GO TO 35
      LUG=LUG+1
      CARRY(LUG)=CARRY(LX)
      LABEL(LUG)=LX
35  CONTINUE
      WRITE(6,106)
      WRITE(6,104) LUG,(LABEL(KJ),KJ=1,LUG)
      PUNCH 105, (CARRY(KM),KM=1,LUG)
      WRITE(6,107) (CARRY(KM),KM=1,LUG)
      NUDE=1
      GO TO 34
30  NUDE=NUDE+1
34  IF(MOUT.NE.NM) GO TO 32
      MOUT=MOUT+1
      GO TO 33
32  CONTINUE
      RETURN
      END

```

\$IBMAP	BCREAD					
	ENTRY	BCREAD		11/1/65	JMLR	1
BCREAD	SAVE	1,4		11/1/65	JMLR	2
	CLA	3,4	GET FIRST ARG.	11/1/65	JMLR	3
	LDQ	4,4	GET SECOND ARG.	11/1/65	JMLR	4
	TLO	*+2	COMPARE	11/1/65	JMLR	5
	XCA		IF 2ND LESS EXCHANGE	11/1/65	JMLR	6
	STQ	TEMP	STORE SMALLEST ARG	11/1/65	JMLR	7
	ADD	SYSONE	ADD 1	11/1/65	JMLR	8
	STA	STD	STORE FOR MOVE	11/1/65	JMLR	9
	SUB	TEMP	COMPUTE COUNT	11/1/65	JMLR	10
	STA	IX1	STORE FOR MOVE	11/1/65	JMLR	11
	AXC	UNC-3,4	LOCATE UNOS LIKE FIV CALL	11/1/65	JMLR	12
	SXA	SYSLOC,4	AND SAVE IN SYSLOC	11/1/65	JMLR	13
	CALL	..BCRD	SET UP READ	11/1/65	JMLR	14
READ	TSX	..FIQC,4	READ RECORD	11/1/65	JMLR	15
	TSX	..FTCK,4	CHECK READ	11/1/65	JMLR	16
IX1	AXT	*+,1	PICK UP COUNT LEFT	11/1/65	JMLR	17
	TXL	LASTC,1,22	IS ONLY 1 REC LEFT	11/1/65	JMLR	18
IX4	AXT	0,4	REC CNT	11/1/65	JMLR	19
	CLA	..BRDB+2,4	MOVE WORDS	11/1/65	JMLR	20
STQ	STQ	*+,1	TO STORE	11/1/65	JMLR	21
	TXI	*+1,1,1	DECR. COUNT	11/1/65	JMLR	22
	TXI	*+1,4,-1	DECR. REC COUNT	11/1/65	JMLR	23
CKIR4	TXH	STD-1,4,-22	CR. REC COUNT	11/1/65	JMLR	24
	SXA	IX1,1	NO SAVE REMAINING COUNT	11/1/65	JMLR	25
	TRA	READ	GO READ NEXT RECORD	11/1/65	JMLR	26
LASTC	TXL	DONE,1,0	ANY MORE WORDS	04/26/66	TS	27
	AXT	DONE,4	YES STORE TO EXIT NEXT	11/1/65	JMLR	28
	SXA	LASTC-1,4	TIME	11/1/65	JMLR	29
	SCD	CKIR4,1	SET REC CNT = NO WORDS LEFT	11/1/65	JMLR	30
	TRA	IX4	GO PROCESS RECORD	11/1/65	JMLR	31
DONE	AXT	READ,4	RESTORE EXIT	11/1/65	JMLR	32
	SXA	LASTC-1,4		11/1/65	JMLR	33
	AXT	-22,4	RESTORE REC CNT	11/1/65	JMLR	34
	SXD	CKIR4,4		11/1/65	JMLR	35
	RETURN	BCREAD		11/1/65	JMLR	36
UNC	PZE	..UNO5.	ADD OF UNIT 5	11/1/65	JMLR	37
TEMP	PZE			11/1/65	JMLR	38
	END					39

```

$18FTC PLOTXY
SUBROUTINE PLOTXY(X,Y,K,P)
C REFERENCE NASA TN D-2174 , APRIL,1964, BY LOIS T. DELLNER
C AND BETTY JO MOORE
C
COMMON/JOLO/N,F,DX,XYX,FORY,STUG,LABOUT,TONLY,KSW64,KPWR,KFD,TLINX
LOGICAL XYX,FORY, STUG,TONLY,XGL,LS
DIMENSION X(1),Y(1),P(1)
DIMENSION FLS(3),FLAB(4),FYLAB(4),YLABEL(11),A(104),ELS(3)
EQUIVALENCE (FLAB(3),IFLAB3),(FYLAB(3),IFYLAB)
DATA MASK1, MASK2,MASK4,MASK8,MASK16,MASK32,MASK64 /
1 O1, O2, O4, O10, O20, O40, O100 /
DATA FYLAB(1),FYLAB(2),FYLAB3,FYLAB(4) /
1 6H(2HP,,6H20X,11,,6H F10.0,1H) /
DATA BLANK,XGRID,YGRID /1H,1H-,1H1 /
DATA RMARK,PCSTD/O726060606060,1H*/
DATA FLS(1),FLS28,FLS264,FLS28,FLS38,FLS38 /
1 6H(2HP+,,4H11X,,3H5X,,3HA5,,3HA6),6H2F6.3) /
DATA FLAB(1),FLAB(2),FLAB3,FLAB(4) /
1 6H(2HP+,,4H17X,,6H F9.0,1H) /
C
100 WRITE (6,500)
500 FORMAT(2HPT)
102 KODE=K
N=P(1)
LABOUT=1
FLAB(3) = FLAB3
LS = .FALSE.
FYLAB(3)=IFYLAB3
KSW8=0
KSW64=0
110 PC=PCSTD
112 IF((AND(KODE,MASK1)).GT.0.)PC = P(2)
114 M=10
116 IF((AND(KODE,MASK2)).GT.0.)M=P(3)
117 IF (M.EQ.0)M=1000
118 NY=10
120 IF((AND(KODE,MASK4)).GT.0.)NY = P(4)
121 IF(NY.EQ.0) NY=1000
122 IF((AND(KODE,MASK64)).GT.0.)KSW64=2
124 IF((AND(KODE,MASK8)).GT.0.)KSW8=1
C
125 K864=KSW8+KSW64
126 IF(K864-2) 132,128,138
128 FLS(2)=FLS264
130 GO TO 139
132 FLS(2)=FLS28
134 FLS(3)=FLS38
136 GO TO 140
138 FLS(2)=FLS28
139 FLS(3)=FLS38
C
140 XYX=.FALSE.
142 FORY=.TRUE.
144 STUG=.FALSE.
146 TONLY=.FALSE.
C
148 IF((AND(KODE,MASK32).LE.0.)) GO TO 172
151 STUG=.TRUE.
152 KSY=P(9)
154 PWR10Y=10.**KSY-6)
156 FY =P(10)*PWR10Y
158 F = FY
C
160 IF(P(5).GE.2.) GO TO 172
162 TONLY=.TRUE.
164 DY= P(11)*PWR10Y
166 DX= DY
172 CALL PISTUG(Y)
173 IF(DX.EQ.0.) GO TO 700
174 FY=F
176 DY=DX
180 IF(KSW64.EQ.2) KPWRY=KPWR
190 IYLAB=IYLAB-KFD
C
200 XYX =.TRUE.
202 FORY=.FALSE.
204 STUG=.FALSE.
206 TONLY=.FALSE.
208 TLINX=55*(1+N/35)
C
210 IF((AND(KODE,MASK16).LE.0.)) GO TO 232
213 STUG=.TRUE.
214 KSX = P(6)
216 PWR10X=010.**KSY-6)
218 FX= P(7)*PWR10X
220 F=FX

```

C	222 IF(MOD(IFIX(P(5)),2).EQ.1) GO TO 232	85
	224 TONLY=.TRUE.	86
	226 DX =P(8)*PWR10X	87
	232 CALL PISTUG(X)	88
	IF(DX.EQ.0.) GO TO 700	89
	234 FX=F	90
	240 IF(KSW64.EQ.2) KPWRX=KPWR	91
	248 IFLAB3=IFLAB3-KFD	92
C		93
	250 IF(KSW64.EQ.0)GO TO 264	94
	252 KOUTX=-KPWRX	95
	254 KOUTY=-KPWRX	96
	256 F10X=10.**KPWRX	97
	258 F10Y=10.**KPWRX	98
	260 WRITE (6,502) KOUTX,KOUTY	99
	502 FORMAT(2HPT,6X, 3HX*E,12,4H Y*E,12)	100
C		101
	264 DO 278 I=1,11	102
	266 TEMP = FY+FLOAT(I-1)*DY*10.	103
	268 ATEMP= ABS(TEMP)	104
	270 IF (ATEMP.LT.1.E-7) TEMP = 0.	105
	272 IF (ATEMP.GE.1.E+7)LABOUT=2	106
	278 YLABEL(I)=TEMP	107
	300 KSYLAB =1	108
	302 WRITE (6,FYLAB) (YLABEL(I),I=1,11)	109
	304 GO TO (306,700),KSYLAB	110
C		111
	306 KSYLAB =2	112
	310 LCTR=0	113
	NCTR=1	114
	KOUT=1	115
	KQUIT= 1	116
C		117
	320 IF(MOD(LCTR,M))328,322,328	118
	322 AFILL= XGRID	119
	324 XGL =.TRUE.	120
	GO TO 330	121
	328 XGL =.FALSE.	122
	AFILL=BLANK	123
	330 DO 332 I=2,104	124
	332 A(I) = AFILL	125
	334 DO 336 I=2,104,NY	126
	336 A(I)= YGRID	127
	A(1) =BLANK	128
	338 GO TO (340,400),KOUT	129
C		130
	340 KX =(X(NCTR)-FX)/DX +.5	131
	342 IF(KX-LCTR)630,350,600	132
	350 KY= (Y(NCTR)-FY)/DY+.5	133
	351 LS= .TRUE.	134
	352 TPC = PC	135
	353 KYL = KY+2	136
	354 IF(KY.LT.0) GO TO 360	137
	356 IF(KY.GT.101)GO TO 364	138
	358 GO TO 370	139
	360 KYL=1	140
	362 GO TO 366	141
	364 KYL=104	142
	366 TPC=RMARK	143
C		144
	370 A(KYL) =TPC	145
	372 J=1	146
	374 IF(KSW8.EQ.0) GO TO 380	147
	376 ELS(J)=P(NCTR+11)	148
	378 J= J+1	149
	380 IF(KSW64.EQ.0) GO TO 386	150
	382 ELS(J)=X(NCTR)/F10X	151
	384 ELS(J+1) = Y(NCTR)/F10Y	152
C		153
	386 IF(NCTR.GE.N)GO TO 392	154
	388 NCTR=NCTR+1	155
	390 GO TO 340	156
C		157
	392 KOUT = 2	158
	394 M= 10	159
		160

C	400 IF(XGL.AND.(MOD(LCTR,10).EQ.0))KQUIT=2	161
		162
C	600 WRITE (6,504)(A(I),I=1,104)	163
	504 FORMAT(2HP,26X,104A1)	164
	602 IF(K864.NE.0) GO TO 620	165
		166
C	604 IF(MOD(LCTR,10))614,606,614	167
	606 XLABEL =FX+FLOAT(LCTR)*DX	168
	608 TEMP =ABS(XLABEL)	169
	610 IF(TEMP.GE.1.E+7)LABOUT=2	170
	612 IF(TEMP.LT.1.E-7)XLABEL=0.	171
	613 WRITE(6,FLAB) XLABEL	172
		173
C	614 LCTR = LCTR+1	174
	616 GO TO (320,302),KQUIT	175
		176
C	620 IF(.NOT.LS) GO TO 604	177
	622 LS = .FALSE.	178
	624 WRITE (6,FLS)(ELS(I),I=1,K864)	179
	626 GO TO 604	180
	630 LABOUT= 4	181
	700 GO TO (710,702,706,704),LABOUT	182
	702 WRITE (6,506)	183
	506 FORMAT(2HPL,3X,10HBAD LABELS)	184
	GO TO 720	185
		186
C	704 WRITE(6,520) NCTR	187
	520 FORMAT(2HPL,18HX OUT OF ORDER. I=,I5)	188
	706 WRITE(6,508)(X(I),Y(I),I=1,2),K,P(1)	189
	508 FORMAT(2HPL,5H N.G.,4G20.8,I6,F8.2)	190
	708 GO TO 720	191
	710 WRITE(6,510)	192
	510 FORMAT(2HPL)	193
	720 RETURN	194
	END	195
		196

\$IBFTC PISTUG ALTIO	1
SUBROUTINE PISTUG(ARRAY)	2
DIMENSION ARRAY(1)	3
COMMON/JOLO/N,F,DX,XYX,FORY,STUG,LABOUT,TONLY,KSH64,KPWR,KFD,TLINX	4
LOGICAL XYX,FORY, STUG,TONLY	5
126 X1 = ARRAY(1)	6
128 IF(XYX) GO TO 133	7
130 DO 132 J = 2,N	8
132 X1 = AMIN1(X1,ARRAY(J))	9
133 IF(STUG) X1=F	10
134 XN = 0.0	11
136 DO 146 J = 1,N	12
138 DIF = ABS(X1-ARRAY(J))	13
140 IF(DIF.LE.XN)GO TO 146	14
142 XN=DIF	15
144 IHQLO=J	16
146 CONTINUE	17
147 XN = ARRAY(IHOLD)	18
148 IF(KSH64.EQ.2) KPWR = KHAR(AMAX1(ABS(X1),ABS(XN)))	19
149 IF(TONLY) GO TO 240	20
150 TLIN=101.	21
152 IF(.NOT.FORY) TLIN = TLINX	22
154 C5 = (XN-X1)/TLIN	23
156 C6 = ABS(C5)	24
158 IF(C6.EQ.0.) GO TO 300	25
159 K7 = KHAR(C6)	26
160 C8 = 10.**K7	27
162 C9 = C6/C8	28
164 IF((2.5-C9).LE.0.0) GO TO 172	29
166 D=2.	30
168 IF((2.0-C9).LE.0.) D=2.5	31
170 GO TO 176	32
172 D=5.	33
174 IF((5.-C9).LE.0.0) D=10.	34
176 C11 = D*C8	35
178 DX = SIGN(C11,C5)	36
179 HUND = 100.*DX	37
240 K7 = KHAR(ABS(DX))	38
250 KFD = 0	39
252 IF(K7) 260,270,254	40
254 IF(K7.GE.5) LABOUT=2	41
256 GO TO 270	42
260 KFD = 6	43
262 IF(K7.LT.(-7)) LABOUT = 2	44
264 IF(K7.GT.(-6)) KFD = -K7	45
270 IF(STUG) GO TO 230	46
182 KC12 = INT(ABS(X1)/C11)	47
184 JJ = 1	48
186 IF(X1) 188,192,190	49
188 JJ = 3	50
190 IF(DX.LT.0.) JJ = JJ+1	51
192 GO TO (193,194,195,196),JJ	52
193 KC14 = KC12	53
GO TO 198	54
194 KC14 = KC12+1	55
GO TO 198	56
195 KC14 = -KC12-1	57
GO TO 198	58
196 KC14 = -KC12	59
198 KC13 = MOD(KC12,10)	60
KC15 = KC12-KC13	61
199 KC18 = KC15	62
200 GO TO (212,202,202,210),JJ	63
202 KC18 = KC18+10	64
204 IF(KC13.NE.9) GO TO 210	65
206 KC18 = KC14	66
208 GO TO 212	67
210 IF(X1.LT.0.)KC18 = -KC18	68
212 F=C11*FLOAT(KC18)	69
214 IF(.NOT.FORY) GO TO 230	70
220 TEMP = F+HUND	71
222 GO TO (224,228,224,228),JJ	72
224 IF(TEMP.GE.XN) GO TO 230	73
226 GO TO 229	74
228 IF(TEMP.LE.XN) GO TO 230	75
229 F=C11*FLOAT(KC14)	76
230 CONTINUE	77
RETURN	78
300 DX=0.	79
LABOUT=3	80
GO TO 230	81
END	
\$IBFTC KHAR ALTIO	1
FUNCTION KHAR(XMAX)	2
KHAR = INT(ALOG(XMAX)/2.302585+40.0)-40	3
RETURN	4
END	

Lewis Research Center,
National Aeronautics and Space Administration,
Cleveland, Ohio, May 10, 1967,
120-27-06-18-22.

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